

663774

DP-1216

AEC RESEARCH AND DEVELOPMENT REPORT

# SAVANNAH RIVER LABORATORY COBALT-60 POWER AND HEAT SOURCES

QUARTERLY PROGRESS REPORT

OCTOBER 1969

SRL  
RECORD COPY



*Savannah River Laboratory*

*Aiken, South Carolina*

## LEGAL NOTICE

This report was prepared as an account of Government sponsored work. Neither the United States, nor the Commission, nor any person acting on behalf of the Commission:

A. Makes any warranty or representation, expressed or implied, with respect to the accuracy, completeness, or usefulness of the information contained in this report, or that the use of any information, apparatus, method, or process disclosed in this report may not infringe privately owned rights; or

B. Assumes any liabilities with respect to the use of, or for damages resulting from the use of any information, apparatus, method, or process disclosed in this report.

As used in the above, "person acting on behalf of the Commission" includes any employee or contractor of the Commission, or employee of such contractor, to the extent that such employee or contractor of the Commission, or employee of such contractor prepares, disseminates, or provides access to, any information pursuant to his employment or contract with the Commission, or his employment with such contractor.

Printed in the United States of America

Available from

Clearinghouse for Federal Scientific and Technical Information  
National Bureau of Standards, U. S. Department of Commerce  
Springfield, Virginia 22151

Price: Printed Copy \$3.00; Microfiche \$0.65

UC-23, Radioisotope and Radiation Applications  
UC-33, Propulsion Systems and Energy Conversion  
(TID-4500, 53rd Ed.)

## **SAVANNAH RIVER LABORATORY COBALT-60 POWER AND HEAT SOURCES**

### **QUARTERLY PROGRESS REPORT**

July - September 1969

Compiled by

C. L. Angerman  
C. P. Ross

November 1969

E. I. DU PONT DE NEMOURS & COMPANY  
SAVANNAH RIVER LABORATORY  
AIKEN, S. C. 29801

*CONTRACT AT(07-2)-1 WITH THE  
UNITED STATES ATOMIC ENERGY COMMISSION*

## PREFACE

This report is one in a series on the applied aspects of  $^{60}\text{Co}$  that are under study at the Savannah River Laboratory (SRL). These reports are intended to present data that are useful to system designers and also to potential or active user agencies. The reports thus deal with the following subject areas of SRL programs:

1. Properties and reactions of  $^{60}\text{Co}$  fuel forms useful or potentially useful as heat sources.
2. Information on the irradiation and postirradiation processing of these materials, when the information is relevant to their use as heat sources and is not in a sensitive area of production technology.
3. Development of design data directed toward the use of and manufacturing capability for isotopic heat sources.

This report contains principally data from work performed during the report period. Previous reports are listed in the Publications section.

### SUMMARY

Incentives and penalties were compared for using oxidized and alloyed forms of radioactive cobalt in heat sources. (p 1)

Melting temperatures were measured for  $\text{CoO}$ ,  $\text{Co}_{0.5}\text{Mg}_{0.5}\text{O}$ , and  $\text{CoAl}_2\text{O}_4$ . (p 14)

Compatibility of the oxidized forms with rhenium was demonstrated at  $1500^\circ\text{C}$ . (p 15)

Capsules of "Inconel" 600 and "Hastelloy" C containing radioactive cobalt metal reached their goal exposure of 10,000 hr at  $900^\circ\text{C}$ . (p 19)

Tungsten capsules containing  $^{60}\text{Co}$  can be handled for short times in air without excessive oxidation, as shown by tests at  $500^\circ\text{C}$ . (p 19)

In preliminary tests, corrosion of cobalt wafers in sea water was more rapid at defects in the nickel plating than rates reported for fully exposed cobalt surfaces. (p 22)

## CONTENTS

	<u>Page</u>
PROGRAM . . . . .	1
MATERIALS TECHNOLOGY AND DEVELOPMENT . . . . .	1
High-Temperature Fuel Candidates . . . . .	1
Comparison of $^{60}\text{Co}$ Fuel Forms . . . . .	1
Fabrication of Oxide Wafers . . . . .	11
Resistance to Thermal Shock . . . . .	11
Melting Temperatures of Cobalt Oxides . . . . .	14
Evaluation of Encapsulation Materials for Radioactive Cobalt . . . . .	15
Compatibility of Cobalt Oxides and Capsule Materials . . . . .	15
Capsule Fabrication and Testing . . . . .	15
Development of Welding Techniques for Refractory Metals . . . . .	15
Heating Tests of Capsules Containing Unirradiated Cobalt Metal . . . . .	16
Heating Tests of Capsules Containing Irradiated Cobalt Metal . . . . .	19
Safety Tests . . . . .	22
HEAT SOURCE DEMONSTRATION TESTS . . . . .	24
WANL 30 kw(t) Demonstration Unit . . . . .	24
HIGH-ACTIVITY $^{60}\text{Co}$ FOR HEAT SOURCE DEVELOPMENT . . . . .	25
SAVANNAH RIVER LABORATORY $^{60}\text{Co}$ PUBLICATIONS . . . . .	26
REFERENCES . . . . .	27

## PROGRAM

The purpose of the Savannah River Laboratory (SRL) program on  $^{60}\text{Co}$  is to provide data that will be required for designing, fabricating, and operating  $^{60}\text{Co}$  heat sources. Primary emphasis is on selecting materials for encapsulating cobalt fuel forms and establishing temperature limits for long-term operation of capsules. Development of specific heat source concepts is not at present included in the scope of the SRL program.

## MATERIALS TECHNOLOGY AND DEVELOPMENT

### HIGH-TEMPERATURE FUEL CANDIDATES

High-melting forms of radioactive cobalt for heat sources are being considered for possible high-temperature applications, including thermionic converters ( $>1500^\circ\text{C}$ ), and for lower temperature applications for which they might provide increased safety over cobalt metal. The fuel forms include  $\text{CoO}$ ,  $\text{CoO-MgO}$  solutions,  $\text{CoAl}_2\text{O}_4$ , and  $\text{Co-Ru}$  and  $\text{Co-Re}$  alloys.

### Comparison of $^{60}\text{Co}$ Fuel Forms

The oxidized and alloyed forms of cobalt have higher melting temperatures than the base metal at the expense of decreased power density. These fuel forms are compared to cobalt metal in Table I.  $\text{CoO}$  and  $\text{MgO}$  form a continuous series of solid solutions, but for purposes of comparison  $\text{CoO-MgO}$  is listed at 50 mol %  $\text{CoO}$ . This composition has a high melting temperature ( $2230^\circ\text{C}$ ) and an acceptable power density (12 w/cc). The metallic alloys,  $\text{Co-Ru}$  and  $\text{Co-Re}$ , are listed at several compositions; alloys with cobalt concentrations less than 35 at. % have power densities less than  $\text{Co}_{0.5}\text{Mg}_{0.5}\text{O}$ , and  $\text{Co-Ru}$  alloys with cobalt concentrations greater than 50 at. % melt at nearly the same temperature as cobalt metal.  $\text{CoAl}_2\text{O}_4$  has a higher melting temperature than  $\text{CoO}$  but its power density is the lowest of all the fuel forms compared. All of the fuel forms except  $\text{Co-Re}$  can be fabricated prior to irradiation;  $\text{Co-Re}$  must be alloyed after irradiation of cobalt because of the high neutron cross section of rhenium and its conversion to osmium, which forms a low-melting alloy with cobalt.

All of the fuels have advantages and disadvantages and none would be optimum for all applications. The intended application, with the corresponding fuel requirements for safety as well as for operation, must be defined to allow selection of a fuel form. Some general conclusions derived from comparing the fuels at  $^{60}\text{Co}$  specific activity of 300 Ci/g are as follows:

TABLE I  
Comparison of  $^{60}\text{Co}$  Fuel Forms

Fuel Form	Co Atom %	Melting Point, °C	Theoretical Density, g/cc	Power Density, <sup>a</sup> w/cc	Advantages	Disadvantages
CoO	50	1800	6.4	23.5	Reasonable power density, fabricate before irradiation. Oxidation resistant	Low thermal conductivity
$\text{Co}_2\text{O}_3\cdot\text{NiO}\cdot 5\text{O}$	25	2230	5.0	12.0	High melting temperature, more stable than CoO, fabricate before irradiation. Oxidation resistant	Lower power density than CoO, low thermal conductivity
$\text{CoAl}_2\text{O}_4$	14.3	1960	4.4	6.9	More stable than CoO, fabricate before irradiation. Oxidation resistant	Very low power density, low thermal conductivity
Co-Ru	20	1990	11.6	6.7	High thermal conductivity, fabricate before irradiation	Very low power density at high melting temperature, low melting temperature at high power density, oxidizes
Co-Ru	30	1860	11.3	10.3		
Co-Ru	40	1730	11.0	14.2		
Co-Ru	50	1600	10.7	18.2		
Co-Re	30	2400	18.1	10.1	High thermal conductivity, good power density	Fabricate after irradiation, oxidizes
Co-Re	40	2170	16.9	13.8		
Co-Re	50	1970	15.8	17.7		
Co-Re	60	1800	14.7	21.9		
Co-Re	70	1670	13.2	26.2		
Co Metal	100	1495	8.8	41.1	High power density, easily fabricated and handled. High thermal conductivity	Lowest melting temperature, oxidizes

<sup>a</sup> $^{60}\text{Co}$  specific activity 300 Ci/g for all forms



- o Power density - The theoretical power density for 100% dense fuel, Figure 1, of CoO is greater than that for  $\text{Co}_{0.5}\text{Mg}_{0.5}\text{O}$ ,  $\text{CoAl}_2\text{O}_4$ , and Co-Ru or Co-Re alloys containing up to 60 at. % cobalt. The power densities of the ruthenium and rhenium alloys are about the same over the full range of cobalt concentrations. The low power density of  $\text{CoAl}_2\text{O}_4$  is a disadvantage that must be balanced against its potential excellent stability at high temperature.
- o Melting temperature - Figure 2, summarizing the melting temperatures, shows  $\text{Co}_x\text{Mg}_{1-x}\text{O}$  and Co-Re to be the most refractory of the fuel forms. The low melting temperature of Co-Ru at reasonable power densities would probably preclude its use unless problems appear in the other fuel forms.
- o Power density and melting temperature - Figure 3 shows the correlation of melting temperature and power density of the fuels. The mixed oxide  $\text{Co}_x\text{Mg}_{1-x}\text{O}$ , CoO, and the alloy Co-Re are superior to the other fuels in this comparison.
- o Operating temperature - Figures 4 and 5, comparing the calculated radial  $\Delta T$ 's in fuel cylinders, show the oxidized forms CoO,  $\text{CoAl}_2\text{O}_4$ , and  $\text{Co}_{0.5}\text{Mg}_{0.5}\text{O}$  to have higher  $\Delta T$ 's than the metallic forms as expected. However, the  $\Delta T$  is only 70°C or less for 1.2-cm-radius cylinders (typical of forms irradiated to date) because of the low internal gamma absorption. Figure 6 compares the maximum surface temperature that can be achieved in cylinders of various radii without melting in the center.  $\text{Co}_{0.5}\text{Mg}_{0.5}\text{O}$  has the highest allowable surface temperature for cylinders through 3 cm in radius. CoO has a lower allowable surface temperature than cobalt metal for cylinders larger than 2 cm in radius.
- o Heat block weight - For the purpose of comparison, fuel cylinders with length equal to diameter were assumed to be surrounded by a tungsten hot shield 2-cm thick to absorb 90% of the gamma radiation escaping from the fuel. Figure 7 shows the weight of the tungsten hot shield as a function of total energy; CoO requires the lightest hot shield because of its higher power density.

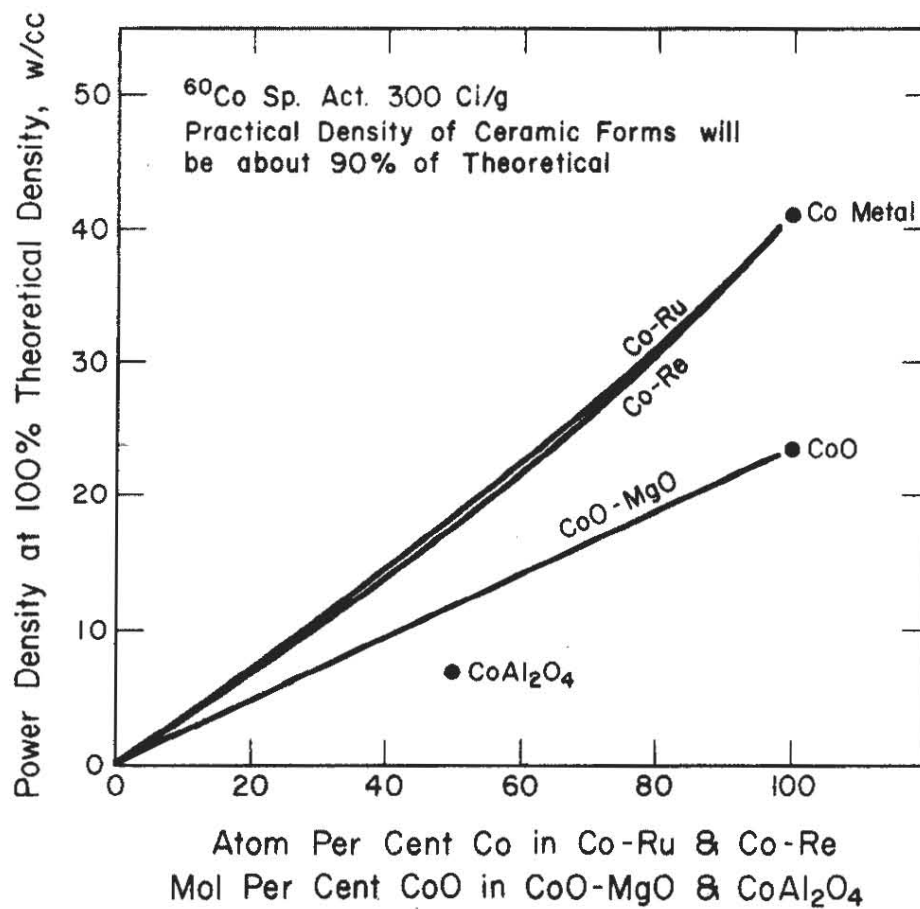


FIG. 1 THEORETICAL POWER DENSITY OF <sup>60</sup>Co FUEL FORMS

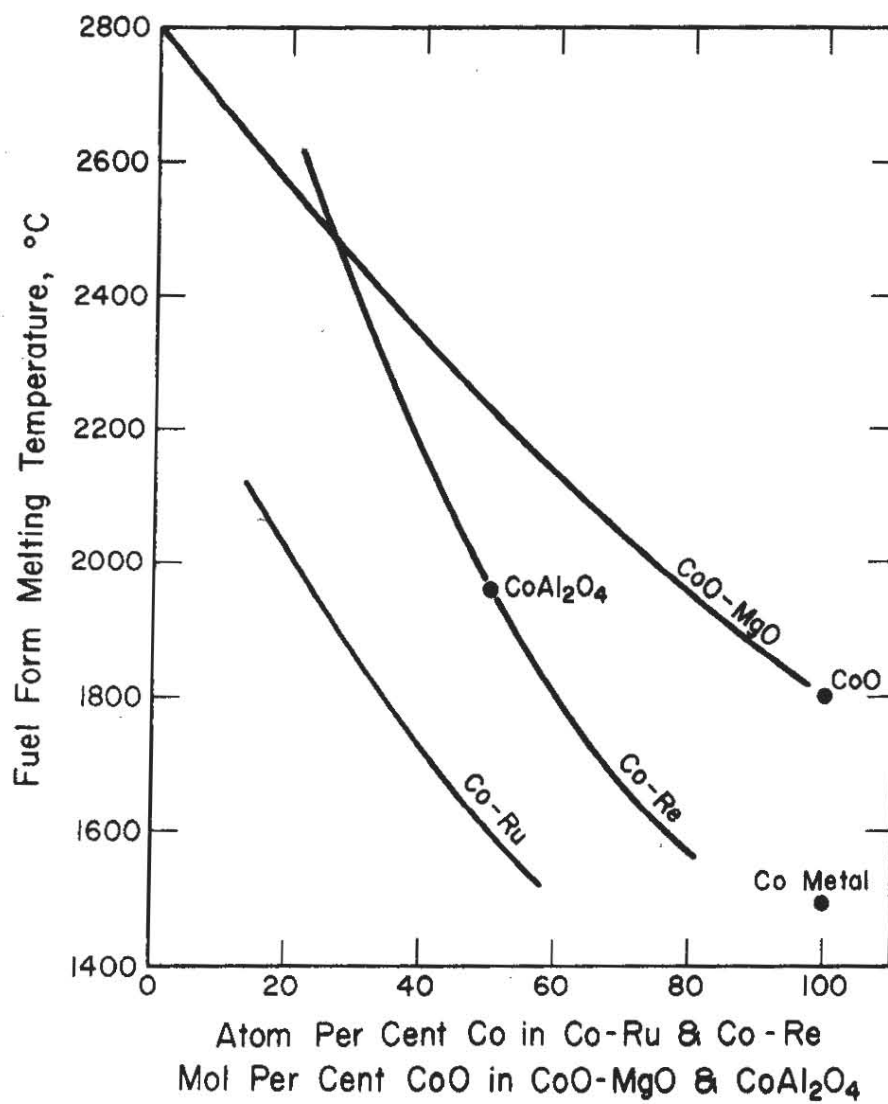


FIG. 2 MELTING TEMPERATURE OF <sup>60</sup>Co FUEL FORMS

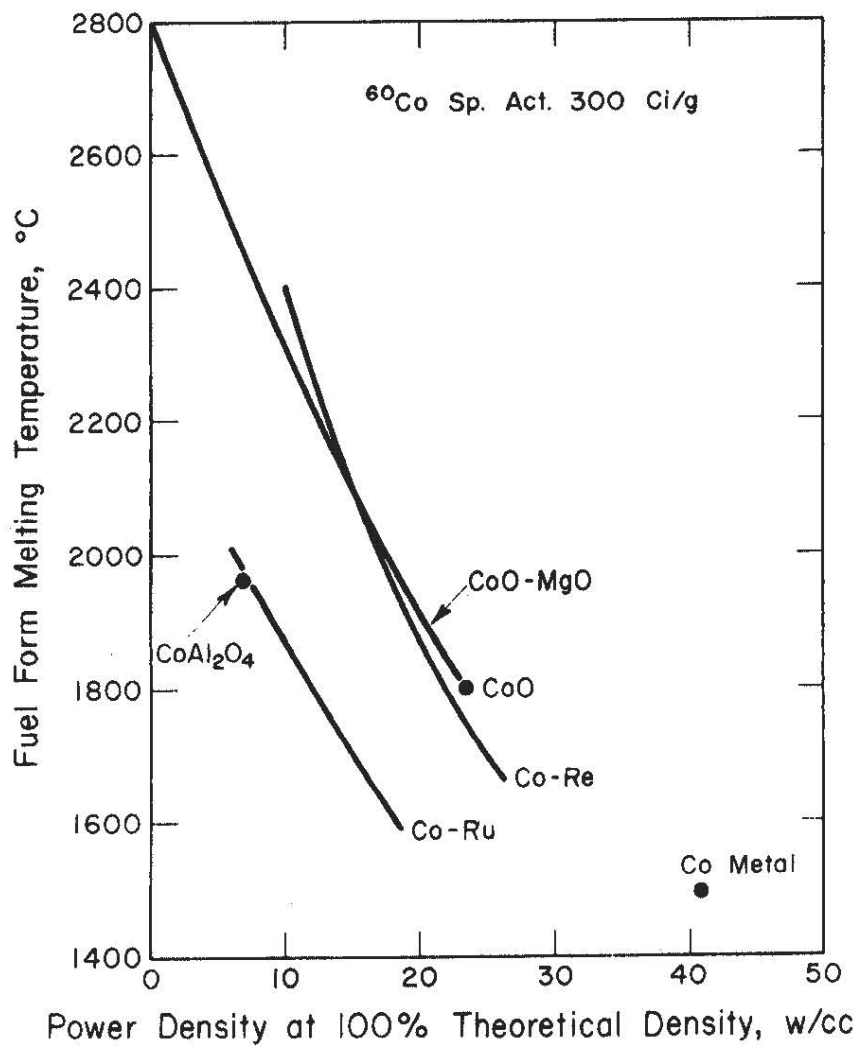


FIG. 3 CORRELATION OF POWER DENSITY AND MELTING TEMPERATURE OF  $^{60}\text{Co}$  FUEL FORMS

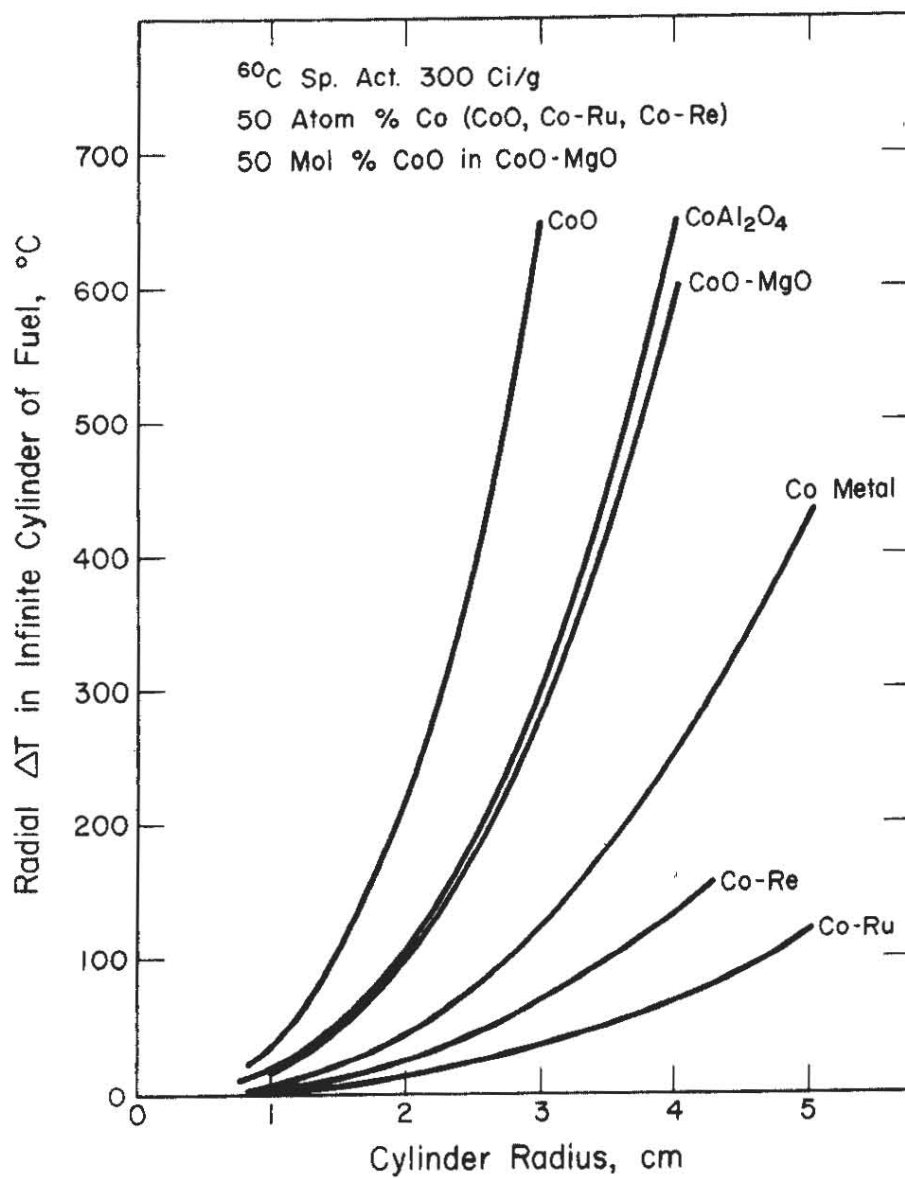


FIG. 4 RADIAL  $\Delta T$  IN INFINITE CYLINDERS OF  $^{60}\text{Co}$  FUEL FORMS

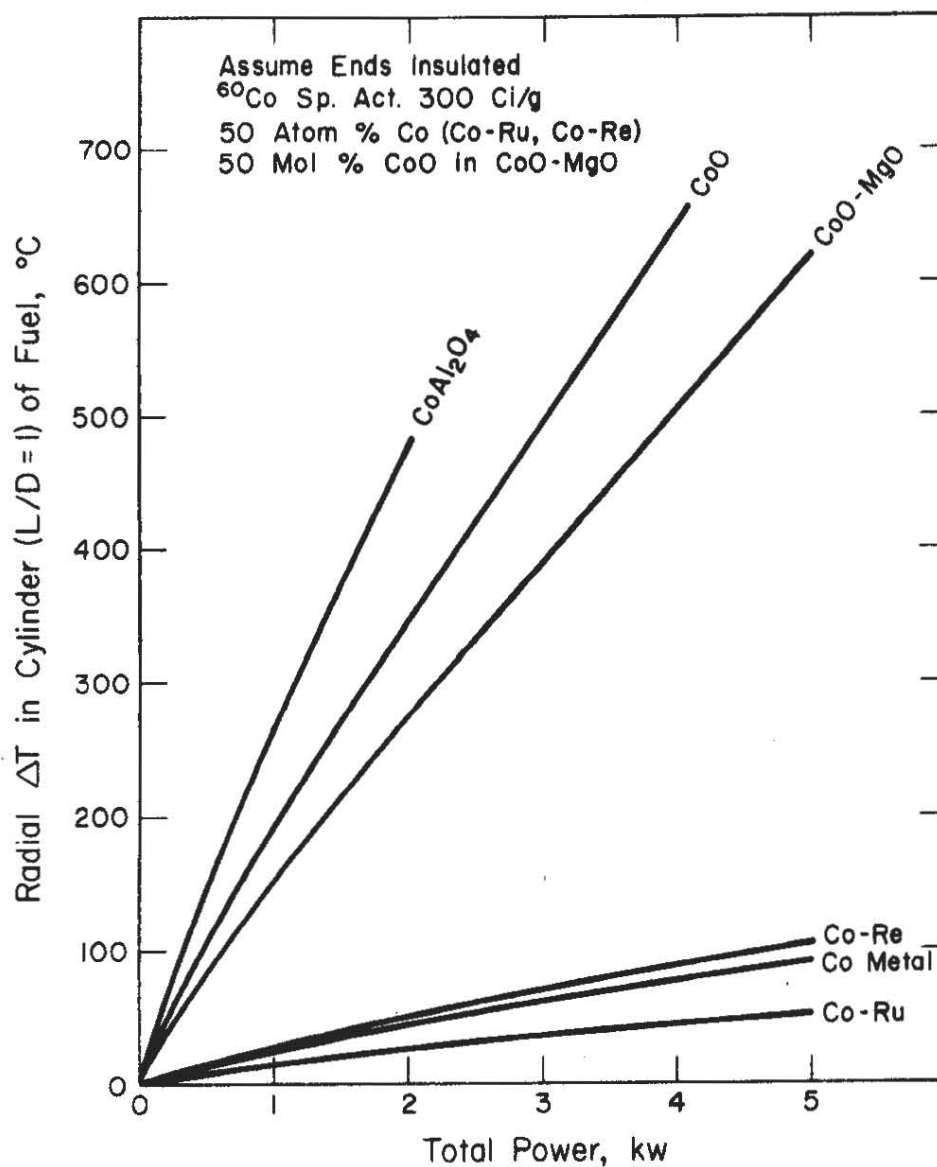


FIG. 5 RADIAL  $\Delta T$  IN FUEL CYLINDERS WITH LENGTH EQUAL TO DIAMETER

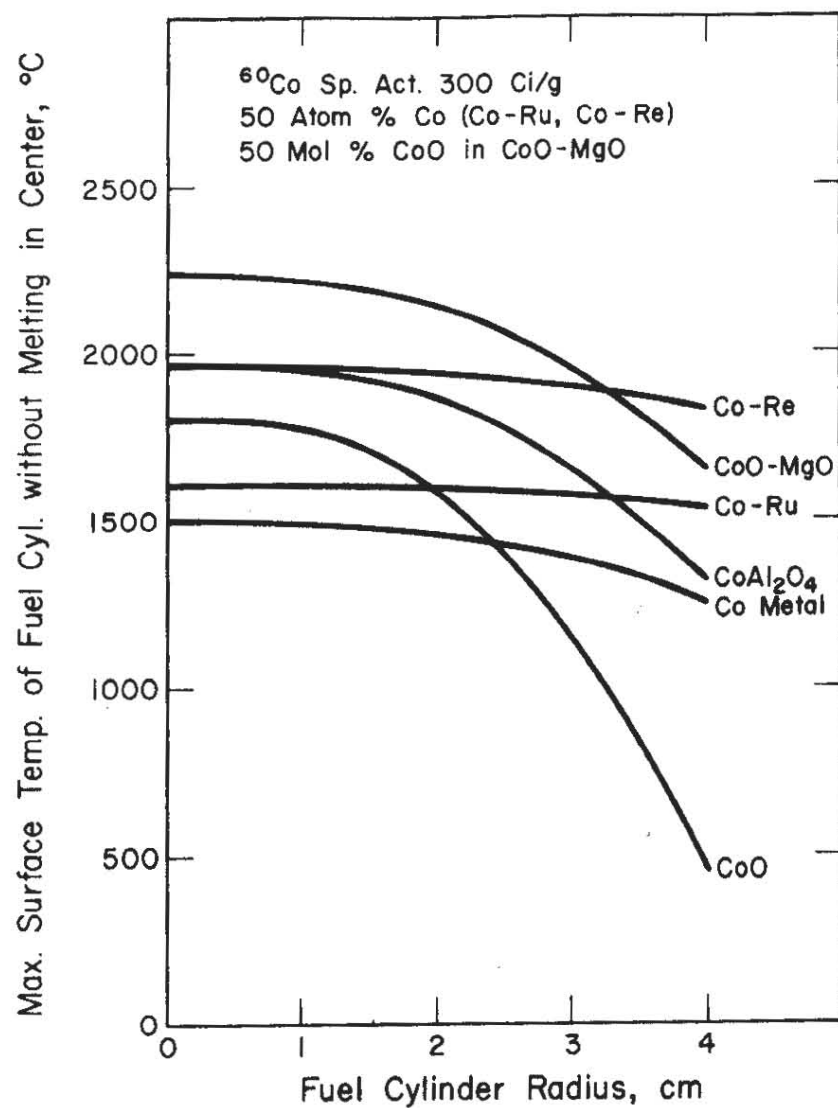


FIG. 6 MAXIMUM SURFACE TEMPERATURE OF FUEL CYLINDER WITHOUT MELTING IN CENTER

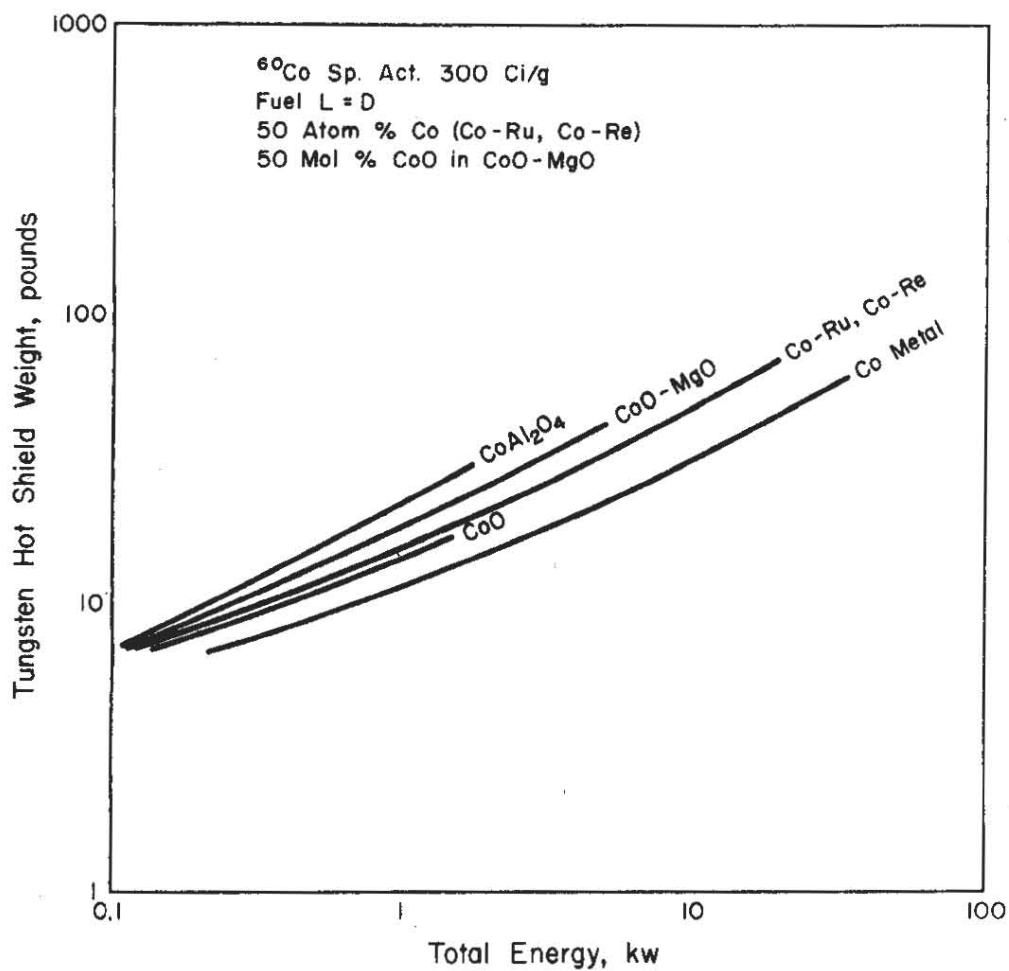


FIG. 7 TUNGSTEN HOT SHIELD TO ABSORB 90% OF GAMMA RADIATION ESCAPING FROM FUEL



- o Overall consideration - From the foregoing conclusions, cobalt metal,  $\text{CoO}$ ,  $\text{Co}_{0.5}\text{Mg}_{0.5}\text{O}$  and Co-Re alloy are probably the best fuel forms for most applications. Final selection would depend on the specific application. SRL work to date has emphasized cobalt metal and the oxide forms because of the difficulty of alloying cobalt with rhenium, especially in a hot cell, and the potential oxidation of the alloy if exposed to air at high temperature.

Because the thermal conductivities of the oxides have not been measured, values for the calculations were estimated from elevated temperature data for similar oxides. A conductivity of 2 pcu/(hr)(ft<sup>2</sup>)(°C/ft) was assumed for  $\text{CoO}$  and  $\text{CoO-MgO}$  and 1 pcu/(hr)(ft<sup>2</sup>)(°C/ft) for  $\text{CoAl}_2\text{O}_4$ .

#### Fabrication of Oxide Wafers

Wafers of  $\text{CoO}$ ,  $\text{Co}_{0.5}\text{Mg}_{0.5}\text{O}$ , and  $\text{CoAl}_2\text{O}_4$  are being fabricated for irradiation in forthcoming high-flux reactor charges and subsequent development of refractory, oxidation resistant heat sources.<sup>1-3</sup> Estimated production in these reactor charges is given in Table II for the ceramic compounds and cobalt metal.

High-density wafers (0.700-in.-dia x 0.150-in.-thick) are fabricated by sintering powders obtained from hydroxide precipitation and decomposition.<sup>4</sup> The procedures followed for each oxide are basically similar and are summarized in the process flow sheet shown in Figure 8.  $\text{CoO}$  wafers are heated for an additional 16 hr in argon at 1000°C to decompose any  $\text{Co}_3\text{O}_4$  that formed during cooling from the sintering temperature.  $\text{Co}_3\text{O}_4$  is not formed in  $\text{Co}_{0.5}\text{Mg}_{0.5}\text{O}$  or  $\text{CoAl}_2\text{O}_4$ .

#### Resistance to Thermal Shock

The oxide wafers were acceptably resistant to cracking when subjected to thermal shocks. Wafers of the three oxides did not crack when cooled from 1100°C to 400°C in 5 seconds. Wafers of  $\text{CoO}$  and  $\text{CoAl}_2\text{O}_4$  did not crack, but wafers of  $\text{Co}_{0.5}\text{Mg}_{0.5}\text{O}$  did, when cooled more rapidly by placing the 1100°C wafers on a polished aluminum disk that was immersed in ice water. Thermal gradients of several hundred degrees were established as the heat drained to the aluminum disk from one side of the wafer.

TABLE II  
Planned Production of High Activity Cobalt  
in  $^{252}\text{Cf}$ -Producing Charges

Cobalt Type <sup>a</sup>	Average Activity, Ci/g Co	No. of Pieces	Total Activity, MCi	Total Power, kw
Metal, 0.745-in.-dia wafers	450	1970	4.08	63.6
	300	1790	2.47	38.5
	200	4560	4.29	66.9
1.00-in.-dia half-wafers	450	396	.74	11.5
	300	286	.36	5.7
1.25-in.-dia half-wafers	450	252	.74	11.5
	300	182	.36	5.7
1.49-in.-dia half-wafers	450	360	1.48	23.0
	300	260	.72	11.4
CoO, 0.700-in.-dia wafers	450	108	.21	3.3
	300	12	.02	0.3
CoO-MgO solid solution, 0.700-in.-dia wafers	450	108	.11	1.7
	300	12	.01	0.2
CoAl <sub>2</sub> O <sub>4</sub> , 0.700-in.-dia wafers	450	108	.06	0.9
	300	12	.01	0.1
TOTAL			15.66	244.3

<sup>a</sup>Metal shapes 0.073-in.-thick  
Ceramic shapes 0.150-in.-thick

#### HYDROXIDE PRECIPITATION

Excess 2.75M NaOH added at 25°C with stirring to:

0.25M  $\text{Co}(\text{NO}_3)_2$  for CoO  
0.25M  $\text{Co}(\text{NO}_3)_2$  and 0.25M  $\text{Mg}(\text{NO}_3)_2$  for  $\text{Co}_{0.5}\text{Mg}_{0.5}\text{O}$   
0.2M  $\text{Co}(\text{NO}_3)_2$  and 0.4M  $\text{Al}(\text{NO}_3)_3$  for  $\text{CoAl}_2\text{O}_4$

#### WASH AND DRY

Wash and centrifuge until pH = 7  
Dry at 120°C; screen - 100 mesh

#### CALCINE

4 hr in flowing argon at 835°C for CoO  
4 hr in flowing argon at 750°C for  $\text{Co}_{0.5}\text{Mg}_{0.5}\text{O}$   
5 hr in flowing argon at 900°C for  $\text{CoAl}_2\text{O}_4$

#### PRESS

Screen through 100 mesh; add binder  
Press at 22,000 psi

#### SINTER

6 hr in air at 1650°C

#### MACHINE

Grind diameter and lap thickness to desired specifications

FIG. 8 PROCESS FLOWSHEET FOR FABRICATION OF COBALT OXIDE WAFERS

## Melting Temperatures of Cobalt Oxides

The melting temperatures of the three oxides were measured by heating samples of the powders used in fabricating wafers for irradiation. Measured values of 1800°C for CoO, 2250°C for  $\text{Co}_{0.5}\text{Mg}_{0.5}\text{O}$ , and 1920°C for  $\text{CoAl}_2\text{O}_4$  agree with values shown on published phase diagrams for the CoO-MgO and CoO- $\text{Al}_2\text{O}_3$  systems, Figures 9 and 10 respectively. CoO and MgO form a continuous series of ideal solid solutions with melting temperatures varying from 1800°C for pure CoO to 2800°C for pure MgO.<sup>5</sup> This system offers a range of power densities and corresponding melting temperatures. The system CoO- $\text{Al}_2\text{O}_3$  contains one compound,  $\text{CoAl}_2\text{O}_4$ , that melts congruently at 1960°C. The adjacent eutectic melting temperatures are estimated at 1910 and 1725°C, depending on whether the system is rich in  $\text{Al}_2\text{O}_3$  or in CoO.

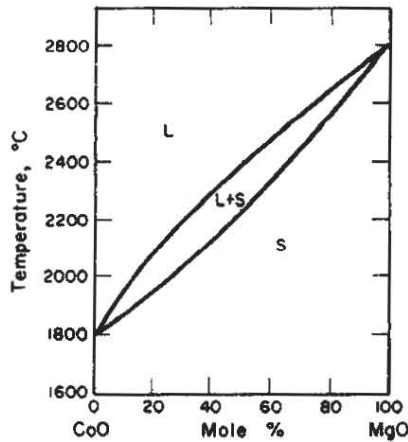


FIG. 9 CoO-MgO PHASE DIAGRAM  
(From Ref. 5)

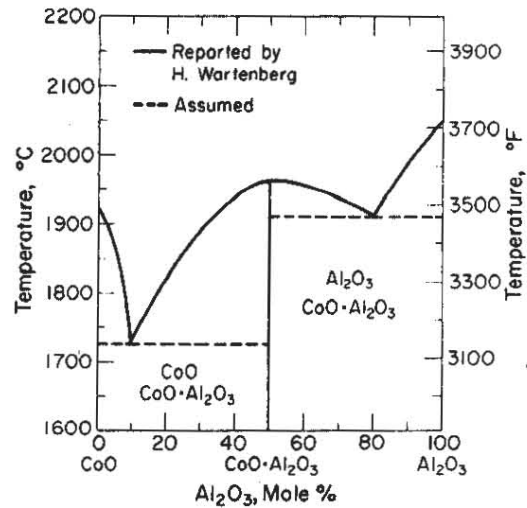


FIG. 10 CoO- $\text{Al}_2\text{O}_3$  PHASE DIAGRAM  
(From Ref. 6)

## EVALUATION OF ENCAPSULATION MATERIALS FOR RADIOACTIVE COBALT

### Compatibility of Cobalt Oxides with Capsule Materials

Thermodynamic analyses of the potential reaction between the three oxides and candidate high-temperature capsule materials indicated that rhenium and the noble metals would be compatible with the oxides.<sup>1</sup> Other refractory metals, such as tantalum, tungsten, and molybdenum, would reduce the oxides to cobalt metal and ternary compounds.

The predicted compatibility of rhenium and the oxides was demonstrated by heating capsules at 1500°C for 200 hr in a vacuum of  $10^{-5}$  torr. Three rhenium capsules were spark-machined from powder-metallurgy rod. Each was loaded with alternating wafers of rhenium and one of the oxides and sealed by TIG welding. After the test, the capsules were intact and unchanged in appearance. Destructive examination showed a uniform black deposit (probably CoO) on the inside wall of each capsule and no gross reaction at the rhenium-oxide interfaces. Detailed metallographic examination and microprobe analyses are in progress.

Short-term compatibility of the oxides with iridium was demonstrated during the measurements of the melting temperature of the oxides. All three oxides melted on iridium filaments at the expected temperatures with no apparent reaction either during the fusion or in subsequent metallographic examination of the iridium filaments. In contrast, an attempt to measure the melting point of CoO on a tungsten filament failed when the tungsten reduced the oxide to cobalt metal at 1400°C; this cobalt then melted at 1495°C, the published value for cobalt metal.

### CAPSULE FABRICATION AND TESTING

#### Development of Welding Techniques for Refractory Metals

Refractory metals and welding equipment are being procured for encapsulation of cobalt metal and cobalt oxides for applications above 1000°C. Capsules of tungsten, tungsten-25% rhenium, and rhenium are being purchased for welding development and long-term compatibility tests at ~1200°C with <sup>59</sup>Co metal. Ten tungsten capsules and five tungsten-25 wt % rhenium alloy capsules have been delivered by the vendor.

Development of welding techniques for tungsten and W-25 wt % Re alloy capsules was delayed pending completion of the inert-gas welding station in the new Alpha Materials Facility (built to support Division of Production  $^{244}\text{Cm}$  and  $^{238}\text{Pu}$  programs). Sealing of the glove box to maintain a helium atmosphere with <10 ppm of  $\text{O}_2$ ,  $\text{N}_2$ , and water vapor as impurities is nearly complete.

## Heating Tests of Capsules Containing Unirradiated Cobalt Metal

### Superalloy Capsules

Four capsules (one "Haynes"\* 25, one "Hastelloy"\* C, and two "Hastelloy"\* X) attained goal exposures of 10,000 hr at 1000°C. Heating of all four capsules was resumed for an additional 10,000 hr because the capsules were intact (helium leak tests) and diameter increases were small, <0.002 in. for the "Hastelloy" capsules and 0.004 to 0.005 in. for the "Haynes" 25 capsule. The larger diameter increase for "Haynes" 25 had been observed in previous 10,000-hr tests. All three materials had demonstrated satisfactory resistance to oxidation and compatibility with cobalt in these earlier tests. These capsules are part of the continuing program to demonstrate structural integrity of superalloy capsules for >10,000 hr at typical temperatures of 850 to 1000°C, Table III.

### Refractory Metal Capsules

A schedule was prepared for long-term heating tests of refractory metal capsules containing unirradiated cobalt metal; these tests will begin in November 1969, Table IV.

---

\*Trademark of Union Carbide Corp.

TABLE III  
Summary of <sup>59</sup>Co Capsule Heating Tests

Capsule Material	Heating		Wall, mils	No. of Capsules	Approx. Starting Date	Approx. Completion Date	Remarks
	Time, hr	Temp, °C					
"Inconel" 600 (m.p. 1370°C)	1,000	850	50	1	12-66	2-67	Capsule intact
	5,000	850	50	1	12-66	7-67	Capsule intact
	10,000	850	50	1	12-66	1-68	Capsule intact
	10,000	850	95	1	7-67	9-68	Capsule intact
	50,000	850	95	1	7-67	3-73	
	1,000	900	95	1	11-68	12-68	Capsule intact
	5,000	900	95	1	11-68	6-69	Capsule intact
	5,000 <sup>a</sup>	900	95	1	3-69	10-69	
	10,000	900	95	1	11-68	1-70	
	10,000	900	95	1	11-68	1-70	
	10,000 <sup>e</sup>	900	95	1	3-69	5-70	
	50,000	900	95	1	11-68	7-74	
	1,000	1,000	50	4	8-66	10-66	3 capsules intact; 1 capsule oxidized <sup>b</sup>
	5,000	1,000	50	1	4-67	11-67	Capsule intact
	1,000 <sup>a</sup>	1,000	95	2	7-67	9-67	No severe oxidation of Co
	1,000 <sup>d</sup>	1,000	95	1	2-68	4-68	No oxidation of Co or capsule
	5,000 <sup>d</sup>	1,000	95	1	2-68	9-68	No oxidation of Co or capsule
	5,000	1,000	95	1	8-67	2-68	Capsule intact
	10,000	1,000	95	1	8-67	10-68	Capsule intact
	50,000	1,000	95	1	10-67	6-73	
	10,000 <sup>d</sup>	1,000	95	1	11-68	1-70	
"Hastelloy" C (m.p. 1270°C)	1,000	1,000	50	4	8-66	10-66	3 capsules intact; 1 capsule oxidized <sup>b</sup>
	5,000	1,000	95	1	10-67	5-68	Capsule intact
	10,000	1,000	95	1	10-67	12-68	Capsule intact
	50,000	1,000	95	1	10-67	6-73	
	20,000	1,000	95	1	5-68	9-70	
TD Nickel (m.p. 1450°C)	1,000	850 <sup>c</sup>	95	1	10-67	12-67	Capsule intact
	5,000	850	95	1	10-67	5-68	Capsule intact
	10,000	850	95	1	10-67	12-68	Capsule intact
	50,000	850	95	1	10-67	6-73	
	1,000	1,000	95	1	12-66	2-67	Capsule intact
	1,000 <sup>a</sup>	1,000	95	2	10-67	12-67	No severe oxidation of Co
TD Nickel Chromium (m.p. 1430°C)	1,000 <sup>a</sup>	1,000	95	2	10-67	12-67	Co near pinhole oxidized
	1,000	1,000	95	1	10-67	12-67	Capsule intact
	5,000	1,000	95	1	10-67	5-68	Capsule intact
	10,000	1,000	95	1	10-67	12-68	Capsule intact
	50,000	1,000	95	1	10-67	6-73	
"Haynes" 25 (m.p. 1330°C)	10,000	850	95	1	11-68	1-70	
	1,000	1,000	95	1	10-67	12-67	Capsule intact
	5,000	1,000	95	1	10-67	5-68	Capsule intact
	5,000	1,000	95	1	5-68	12-68	Capsule intact
	10,000	1,000	95	1	10-67	12-68	Capsule intact
	50,000	1,000	95	1	10-67	6-73	
"Hastelloy" X (m.p. 1260°C)	20,000	1,000	95	1	5-68	9-70	
	1,000	1,000	50	1	4-67	6-67	Capsule intact
	5,000	1,000	50	1	4-67	11-67	Capsule intact
	5,000	1,000	95	2	2-68	9-68	Capsules intact
	10,000	1,000	95	1	2-68	4-69	Capsule intact
	50,000	1,000	95	1	2-68	10-73	
	20,000	1,000	95	2	5-68	9-70	

<sup>a</sup>Two capsules, one not welded and one with drilled hole in wall, to test effects of capsule defects.

<sup>b</sup>Capsules reacted with fire-brick. See DP-1094, "SRI Isotopic Power and Heat Sources - Quarterly Progress Report," October-December 1966.

<sup>c</sup>Tests of TD Nickel at 850°C in flowing argon.

<sup>d</sup>Internal atmosphere air instead of helium.

<sup>e</sup>Caustic residue on wafers.

TABLE IV  
Summary of Planned <sup>59</sup>Co-Refractory Metal Capsule Heating Tests<sup>a</sup>

Capsule Material	Heating		Approx. Starting Date	Approx. Completion Date	Wafer Diameter, inch	Welding Technique
	Time, hr	Temp, °C				
Tungsten	1,000	1200	11-69	1-70	0.745	TIG
	1,000	1200	11-69	1-70	0.745	TIG
	5,000	1200	11-69	6-70	0.745	TIG
	5,000	1200	11-69	6-70	0.745	TIG
	5,000	1200	2-70	9-70	1.490	TIG
	5,000	1200	3-70	10-70	0.745	EB
	10,000	1200	11-69	1-71	0.745	TIG
	10,000 <sup>b</sup>	1200	12-69	2-71 <sup>b</sup>	0.745	TIG
	10,000	1200	2-70	4-71	1.490	TIG
	10,000	1200	2-70	4-71 <sup>b</sup>	1.490	TIG
	10,000	1200	3-70	5-71	0.745	EB
	50,000	1200	12-69	8-75	0.745	TIG
Rhenium	1,000	1200	12-69	2-70	0.745	TIG
	5,000	1200	12-69	7-70	0.745	TIG
	5,000	1200	3-70	10-70	0.745	EB
	10,000	1200	1-70	3-71	0.745	TIG
	10,000 <sup>b</sup>	1200	1-70	3-71 <sup>b</sup>	0.745	TIG
	10,000	1200	3-70	5-71	0.745	EB
	50,000	1200	1-70	9-75	0.745	TIG
	1,000	1400	2-70	4-70	0.745	TIG
	5,000	1400	2-70	9-70	0.745	TIG
	10,000	1400	2-70	4-71	0.745	TIG
W-25 wt % Re	1,000	1200	11-69	1-70	0.745	TIG
	5,000	1200	12-69	7-70	0.745	TIG
	5,000	1200	3-70	10-70	0.745	EB
	10,000	1200	1-70	3-71	0.745	TIG
	10,000 <sup>b</sup>	1200	1-70	3-71 <sup>b</sup>	0.745	TIG
	10,000	1200	3-70	5-71	0.745	EB
	50,000	1200	1-70	9-75	0.745	TIG
	1,000	1400	2-70	4-70	0.745	TIG
	5,000	1400	2-70	9-70	0.745	TIG
	10,000	1400	2-70	4-71	0.745	TIG

<sup>a</sup>One capsule containing ~10 Co wafers 0.073-in.-thick and one 0.060-in.-thick spacer will be heated at each listed condition.

<sup>b</sup>Test time may be extended to 20,000 hr, or more, depending on results of other 10,000-hr tests.



## Heating Tests of Capsules Containing Irradiated Cobalt Metal

### Superalloy Capsules

Fifteen superalloy capsules, 13 containing irradiated and 2 containing unirradiated cobalt metal, are being heated in air at 900 to 1000°C in the High Level Caves (HLC) to demonstrate capsule performance at typical heat source conditions, Table V. An "Inconel"\* 600 capsule containing 36,500 Ci and a "Hastelloy" C capsule containing 13,000 Ci achieved their goal exposures of 10,000 hr at 900°C and were removed from test. Another "Inconel" 600 capsule containing 13,700 Ci also reached 10,000 hr at 900°C; heating of this capsule will be continued for an additional 10,000 hr unless examination of the first "Inconel" capsule reveals unsatisfactory performance. Destructive examination of these two capsules and the three removed from test in April after 5000 hr at 1000°C are scheduled to begin in January 1970 after completion of the rearrangement of High Level Cave metallographic facilities.

### Refractory Metal Capsules

A schedule was prepared for long-term heating tests of refractory metal capsules containing irradiated cobalt metal; these tests will begin in January 1971, Table VI.

### Test Facilities

Design of a shielded capsule test facility for the Isotope Process Development Laboratory (IPDL) is continuing. Preparation of a Project Authorization Request for operating and process equipment for the facility was started.

### Oxidation of Tungsten

A study of the oxidation of tungsten in air showed that relatively simple precautions will be required to prevent serious oxidation of tungsten capsules during fabrication and handling prior to the long-term heating tests in high-vacuum furnaces. Cooling the capsules below 500°C should provide adequate protection since the maximum depth of oxidation was <0.001 in. when a tungsten sample was heated in air at 500°C for 6 hr. Oxidation extended to a depth of ~0.011 in. after heating for 2 hr at 900°C, the maximum calculated surface temperature for a 1-in.-dia x 1-in.-long capsule with an emissivity of 0.2 and containing 15,000 Ci of  $^{60}\text{Co}$  or ~250 w(t). Thermal cycling between room

\*Trademark of International Nickel Co.

temperature and 900°C for 2 hr (20 min at 900°C/cycle) affected approximately twice this depth. Oxidation was uniform at both temperatures; no pitting or preferential attack occurred at the grain boundaries. Additional tests will be made at ~600°C, the expected capsule temperature (emissivity of 0.6).

TABLE V  
Summary of  $^{60}\text{Co}$  Capsule Heating Tests

Capsule Material	Heating		Wall, mils	No. of Capsules	Activity		Approx. Starting Date	Approx. Completion Date	Remarks
	Time, hr	Temp, °C			Spec. Ci/g	Total, Ci			
"Inconel" 600 (m.p. 1370°)	130	850 <sup>a</sup>	50	1	120	16,000	2-67	2-67	Swelled due to overheating
	1,000	~900	50	1	100	5,000	4-67	6-67	Capsule intact
	5,000	~900	50	1	150 <sup>b</sup>	15,000	4-67	10-67	Capsule intact
	10,000	~900	50	1	150 <sup>b</sup>	15,000	4-67	6-68	Increased Co/capsule reaction
	10,000	~900	50	1	150 <sup>b</sup>	9,000	5-67	10-68	Increased Co/capsule reaction
	+10,000	900	95	1	255 <sup>c</sup>	36,500	2-68	8-69	Begin destructive examination 1-70
	+20,000	900	95	1	288 <sup>c</sup>	13,700	7-68	10-70	
	50,000	900	95	1	282 <sup>c</sup>	13,400	7-68	3-74	
	5,000	1,000	95	1	295 <sup>c</sup>	14,000	9-68	4-69	Begin destructive examination 1-70
	10,000	1,000	95	1	288 <sup>c</sup>	13,700	9-68	11-69	
	10,000 <sup>d</sup>	1,000	95	1	263 <sup>c</sup>	12,500	9-68	11-69 <sup>e</sup>	
	50,000	1,000	95	1	255 <sup>c</sup>	12,100	9-68	5-74	
	10,000	850	95	1	d	-	9-68	11-69	
"Hastelloy" C (m.p. 1270°C)	100	850	50	1	120	9,000	1-67	1-67	Capsule intact
	+10,000	900	95	1	276 <sup>c</sup>	13,100	7-68	8-69	Begin destructive examination 1-70
	10,000	1,000	95	1	282 <sup>c</sup>	13,400	9-68	11-69	
	50,000	1,000	95	1	270 <sup>c</sup>	12,800	9-68	5-74	
"Haynes" 25 (m.p. 1330°C)	5,000	1,000	95	1	263 <sup>c</sup>	12,500	9-68	4-69	Begin destructive examination 1-70
	10,000	1,000	95	1	288 <sup>c</sup>	13,700	9-68	11-69	
	10,000 <sup>d</sup>	1,000	95	1	282 <sup>c</sup>	13,400	9-68	11-69 <sup>e</sup>	
	50,000	1,000	95	1	295 <sup>c</sup>	14,000	9-68	5-74	
	10,000	850	95	1	d	-	9-68	11-69	
"Hastelloy" X (m.p. 1260°C)	5,000	1,000	95	1	250 <sup>c</sup>	11,900	9-68	4-69	Begin destructive examination 1-70
	10,000	1,000	95	1	263 <sup>c</sup>	12,500	9-68	11-69	
	10,000 <sup>d</sup>	1,000	95	1	263 <sup>c</sup>	12,500	9-68	11-69 <sup>e</sup>	
	50,000	1,000	95	1	301 <sup>c</sup>	14,300	9-68	5-74	

<sup>a</sup> Excursion to >1100°C for 3-6 hr.

<sup>b</sup> Activity as of 2-67.

<sup>c</sup> Activity as of 6-68.

<sup>d</sup> Capsule contains  $^{59}\text{Co}$  but is being heated along with  $^{60}\text{Co}$  capsules.

<sup>e</sup> New information reported

TABLE VI

Summary of Planned  $^{60}\text{Co}$ -Refractory Metal Capsule Heating Tests<sup>a</sup>

Capsule Material	Heating		Approx. Starting Time	Approx. Completion Date	Wafer Diameter, inch
	Time, hr	Temp, °C			
Tungsten	1,000	1200	1-71	3-71	0.745
	1,000	1200	1-71	3-71	0.745
	5,000	1200	1-71	8-71	0.745
	5,000	1200	1-71	8-71	0.745
	5,000	1200	3-71	10-71	1.490
	10,000	1200	1-71	3-72	0.745
	10,000 <sup>b</sup>	1200	1-71	3-72 <sup>b</sup>	0.745
	10,000	1200	3-71	7-72	1.490
	10,000 <sup>b</sup>	1200	3-71	7-72	1.490
	50,000	1200	1-71	9-76	0.745
Rhenium	5,000	1200	2-71	9-71	0.745
	10,000	1200	2-71	4-72	0.745
	10,000 <sup>b</sup>	1200	2-71	4-72 <sup>b</sup>	0.745
	50,000	1200	2-71	10-76	0.745
	5,000	1400	3-71	7-72	0.745
	10,000	1400	3-71	7-72	0.745
	10,000 <sup>b</sup>	1400	3-71	7-72 <sup>b</sup>	0.745
	50,000	1400	3-71	11-76	0.745
W-25 wt % Re	5,000	1200	2-71	9-71	0.745
	10,000	1200	2-71	4-72	0.745
	10,000 <sup>b</sup>	1200	2-71	4-72 <sup>b</sup>	0.745
	50,000	1200	2-71	10-76	0.745
	5,000	1400	3-71	10-71	0.745
	10,000	1400	3-71	7-72	0.745
	10,000 <sup>b</sup>	1400	3-71	7-72 <sup>b</sup>	0.745
	50,000	1400	3-71	11-76	0.745

<sup>a</sup>One capsule (ten wafers 0.073-in.-thick and one spacer 0.060-in.-thick) will be heated at each listed condition. Capsules with 0.745-in.-dia wafers will contain ~250 watts, or 16,000 Ci of  $^{60}\text{Co}$ ; capsules with larger wafers will produce up to 1000 watts.

<sup>b</sup>Test time may be extended to 20,000 hr, or more, depending on results of other 10,000-hr tests.

## Safety Tests

### Cobalt Release

Measurement of the release of  $^{60}\text{Co}$  from an intentionally defected "Inconel" 600 capsule heated in flowing air at  $1000^{\circ}\text{C}$  is continuing.<sup>2</sup> The capsule has a 0.008-in.-dia hole drilled through the wall and contains a solid rod of unirradiated cobalt electroplated with  $^{60}\text{Co}$ . Low levels of radioactivity ( $\sim 10$  c/m above background) become deposited each day on a filter located in the effluent air stream. Total test time to date is  $\sim 4000$  hr.

### Corrosion of Cobalt Metal in Sea Water

Six unirradiated, nickel-plated cobalt wafers were immersed for 865 hr in sea water (North Carolina coast) in preliminary tests to measure their corrosion resistance. The wafers, held 1/4-in. apart in a "Teflon"\* rack, were suspended from a piling to a depth of 3 to 24 in., depending on tide level. Three of the wafers were mechanically dislodged by wave action and were lost.

Severe corrosion of the cobalt occurred wherever there was a defect in the nickel coating. The attack spread laterally around a defect, leaving the nickel coating intact over most of the corroded area; in two of the wafers the corrosion penetrated the 0.040-in.-thick wafers, Figure 11. Approximately 0.09 g of cobalt was lost from each wafer. This rate is apparently faster than the 0.0007 in./yr ( $0.1 \text{ g/yr/in}^2$  of surface area) that was observed after 4-yr tests with panels cut from cobalt electrodes.<sup>7</sup> A direct comparison of the two tests is difficult because the exposed surface area of the cobalt wafers was constantly changing. The small size of the defects in the nickel coating ( $< 0.005$ -in.-dia) may have restricted the supply of oxygen to the corroding surfaces, leading to accelerated attack, as in crevice corrosion.

\*Trademark of E. I. du Pont de Nemours & Co.

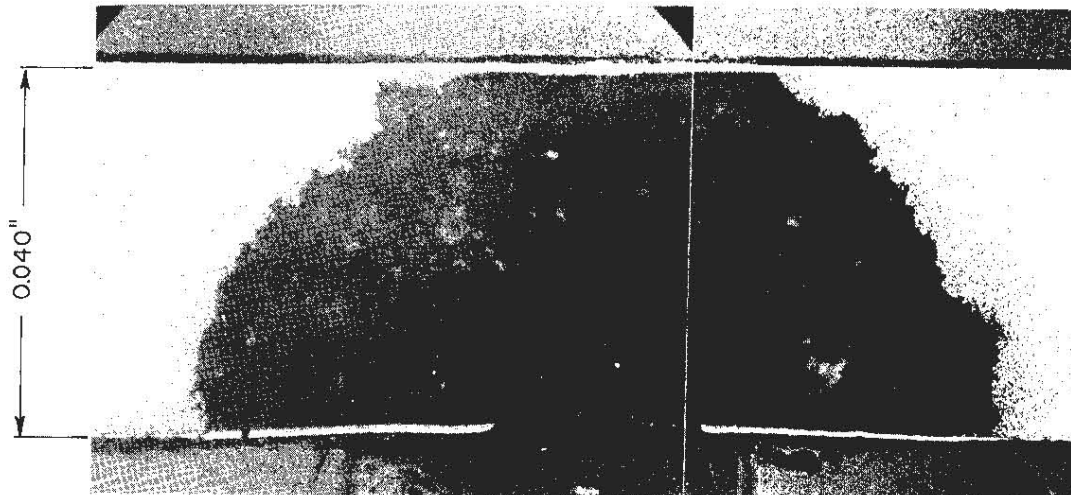


FIG. 11 SEAWATER CORROSION OF COBALT AT DEFECT  
IN NICKEL COATING

Recommended limits on release rates of radioactive wastes from nuclear powered ships during normal operation range from 0.01 to 100 Ci/day, depending on closeness to shore and ocean depth.<sup>8</sup> For a specific activity of 350 Ci/g, corrosion under the abnormal conditions of the current tests could release ~1.0 Ci of  $^{60}\text{Co}$ /day. Additional tests of the effects of sea water corrosion are being planned.

#### Compatibility of Molten Cobalt and Superalloys with Tungsten and Rhenium

Tests of compatibility of molten cobalt and superalloys with tungsten and rhenium will be continued to evaluate the consequences of temperature excursions above the melting points of the capsules ( $\sim 1500^\circ\text{C}$ ) that might occur under abnormal conditions, such as loss of coolant, re-entry from space, or burial in the earth.<sup>2</sup>

Five additional tests with tungsten capsules will be started as soon as the new welding facility is completed (see Development of Welding Techniques for Refractory Metals). These capsules will be heated for 4 hr at 1550 to 1750°C to evaluate the effects of a void space and superalloys inside the tungsten capsule and the effectiveness of a rhenium barrier between the cobalt and the tungsten.

## HEAT SOURCE DEMONSTRATION TESTS

### WANL 30 kw(t) Demonstration Unit

SRL is providing technical assistance as necessary in the program to design, develop, fabricate, and test an experimental  $^{60}\text{Co}$ -fueled 30 kw(t) heat source. Westinghouse Astronuclear Laboratory (WANL) is the contractor in this program to demonstrate the principle of production of useful energy from a large  $^{60}\text{Co}$  source.

Tests were planned to verify calculations of radiation streaming through ducts in the biological shield. These tests will be run at SRL, using an available 0.115 curie source of  $^{60}\text{Co}$  and  $\text{CaF}_2$  thermoluminescent detectors. The shield block for the test is scheduled for delivery in December.

Nickel was selected by WANL as the reference core material because of its high thermal conductivity, and nitrogen as the preferred coolant for compatibility at up to  $900^\circ\text{C}$ . Work is continuing to evaluate air as a possible coolant, and SRL is participating in a survey of coatings for oxidation protection of the nickel.

# HIGH ACTIVITY $^{60}\text{Co}$ FOR HEAT SOURCE DEVELOPMENT

High activity cobalt that is being stored at Savannah River for experimental programs and heat source development is listed in Table VII. Estimated production in current charges is listed in Table II.

TABLE VII  
 $^{60}\text{Co}$  for Heat Sources<sup>a</sup>

Cobalt Shape	Average Activity, Ci/g	No. of Pieces	Total Activity, $10^6$ Ci	Total Power, kw
<u>0.745-in. Wafers</u>				
Fuel form - cobalt metal (wrought)	310	2090	1.60	25.0
Fuel shape - wafers 0.745 $\pm$ 0.001-in.-dia	270	912	0.62	9.6
0.040 $\pm$ 0.003-in.-thick	250	1368	0.87	13.6
including 0.0005 to 0.001-in. Ni plate	230	1368	0.79	12.3
Cobalt density - 8.80 $\pm$ 0.05 g/cm <sup>3</sup>	240	152	0.09	1.4 <sup>b</sup>
		5890	3.97	61.9
<u>0.800-in. Wafers</u>				
Fuel form - cobalt metal (sintered)	300	255	0.22	3.4
Fuel shape - wafers 0.800 $\pm$ 0.001-in.-dia	320	136	0.13	2.0 <sup>c</sup>
0.040 $\pm$ 0.003-in.-thick	270	136	0.10	1.6 <sup>d</sup>
including 0.0005 to 0.001-in. Ni plate		527	0.45	7.0
Cobalt density - 8.60 $\pm$ 0.10 g/cm <sup>3</sup>				
<u>Nickel-Plated Slabs</u>				
Fuel form - cobalt metal (wrought)	300	45	0.23	3.6
Fuel shape - slabs 3.00 $\pm$ 0.03-in.-long	280	48	0.22	3.4
0.640 $\pm$ 0.002-in.-wide		93	0.45	7.0
0.060 $\pm$ 0.001-in.-thick				
including 0.0005 to 0.001-in. Ni plate				
Cobalt density - 8.80 $\pm$ 0.05 g/cm <sup>3</sup>				
<u>Stainless Steel-Canned Slabs</u>				
Fuel form - cobalt metal (wrought)	300	45	0.18	2.8
Fuel shape - slabs 2.96 $\pm$ 0.03-in.-long	250	48	0.17	2.7
0.735 - 0.740-in.-wide		93	0.35	5.5
0.092 $\pm$ 0.001-in.-thick				
sheath thickness 0.015-in. min				
cobalt dimensions same as nickel-plated slabs above except 2.44-in.-long				
Cobalt density - 8.80 $\pm$ 0.05 g/cm <sup>3</sup>				
<u>Stainless Steel-Coextruded Slabs</u>				
Fuel form - cobalt metal (wrought)	300	60	0.21	3.3
Fuel shape - slabs 3.00 $\pm$ 0.03-in.-long	280	64	0.21	3.3
0.740 $\pm$ 0.002-in.-wide		124	0.42	6.6
0.072 $\pm$ 0.002-in.-thick				
SST thickness 0.015-in. min				
Cobalt dimensions: 2.75-in.-long				
0.71-in.-wide				
0.042-in.-thick				
Cobalt density - 8.80 $\pm$ 0.05 g/cm <sup>3</sup>				
Grand Total			5.64 MCi	88.0 kw
Average Activity			280 Ci/g	

<sup>a</sup> Activity and power as of June 30, 1969.

<sup>b</sup> Wafers have central hole of 0.070-in.-dia.

<sup>c</sup> 110 wafers have experimental compositions, to be used at SRL.

<sup>d</sup> 68 wafers have central hole of 0.070-in.-dia.

## SAVANNAH RIVER LABORATORY $^{60}\text{Co}$ PUBLICATIONS

### QUARTERLY PROGRESS REPORTS

"Savannah River Laboratory Isotopic Power and Heat Sources Quarterly Progress Report," compiled by H. S. Hilborn

DP-1088	July - September 1966
DP-1094	October - December 1966
DP-1105-I	January - March 1967, Part I - Cobalt
DP-1120-I	April - June 1967, Part I - Cobalt
DP-1129-I	July - September 1967, Part I - Cobalt
DP-1143-I	October - December 1967, Part I - Cobalt
DP-1155-I	January - March 1968, Part I - Cobalt
DP-1169-I	April - June 1968, Part I - Cobalt
DP-1177-I	July - September 1968, Part I - Cobalt
DP-1192-I	October - December 1968, Part I - Cobalt
DP-1196-I	January - March 1969, Part I - Cobalt
DP-1206-I	April - June 1969, Part I - Cobalt

### TOPICAL REPORTS

DP-974	" $^{60}\text{Co}$ Heat Sources for 10-60 kw(e) Generators" by A. H. Dexter, July 1965.
DP-1012	"Radioactive Cobalt for Heat Sources" by J. W. Joseph, H. F. Allen, C. L. Angerman, and A. H. Dexter, October 1965.
DP-1051 (Rev. 2)	"Properties of $^{60}\text{Co}$ and Cobalt Metal Fuel Forms", June 1968.
DP-1096	"Development of $^{60}\text{Co}$ Capsules for Heat Sources" by C. P. Ross, C. L. Angerman, and F. D. R. King, June 1967.
DP-1145	"Experimental $^{60}\text{Co}$ Heat Source Capsules" by J. P. Faraci, May 1968.

### JOURNAL ARTICLES

A. H. Dexter, W. R. Cornman, and E. J. Hennelly. "The Advantages of  $^{60}\text{Co}$  for Heat and Radiation Sources", *Nucl. Appl.* 2(2), 99-101 (1966).

C. P. Ross. "Cobalt-60 for Power Sources", *Isotopes and Radiation Technology*, 5(3), 185-94 (1968).

C. L. Angerman, F. D. R. King, J. P. Faraci, and A. E. Symonds. " $^{60}\text{Co}$  Heat Source Encapsulation", *Nucl. Appl.* 4(2), 88-95 (1968).



C. L. Angerman and J. P. Faraci. "Heating Tests of Encapsulated Cobalt Heat Sources", *Nuclear Metallurgy, Vol. 14, Symposium on Materials for Radio-Isotope Heat Sources*, D. E. Thomas, W. O. Harms, and R. T. Huntoon (Editors), American Institute of Mining, Metallurgical, and Petroleum Engineers, Inc., New York, New York, pages 309-22 (1969).

## REFERENCES

1. H. S. Hilborn (Compiler). Savannah River Laboratory Isotopic Power and Heat Sources Quarterly Progress Report, October-December 1968, Part I - Cobalt-60. USAEC Report DP-1192-I, E. I. du Pont de Nemours and Co., Savannah River Laboratory, Aiken, S. C. (1969).
2. H. S. Hilborn (Compiler). Savannah River Laboratory Isotopic Power and Heat Sources Quarterly Progress Report, January - March 1969, Part I - Cobalt-60. USAEC Report DP-1196-I, E. I. du Pont de Nemours and Co., Savannah River Laboratory, Aiken, S. C. (1969).
3. C. L. Angerman and C. P. Ross. Savannah River Laboratory Isotopic Power and Heat Sources Quarterly Progress Report, April - June 1969, Part I - Cobalt-60. USAEC Report DP-1206-I, E. I. du Pont de Nemours and Co., Savannah River Laboratory, Aiken, S. C. (1969).
4. P. K. Smith, J. R. Keski, P. E. McBeath, W. R. Kanne, and W. C. Mosley. "Cobalt-60 Oxides as Thermionic Fuels". To be presented at the Thermionic Conversion Specialist Conference, October 22, 1969 and to be published in the Proceedings of the conference.
5. H. von Wartenburg and E. Prophet. "Melting-Point Diagrams of Refractory Oxides: V Systems with Magnesia." *Z. anorg. u. allgem. Chem.*, 208, 369 (1932).
6. H. von Wartenburg and H. J. Reusch. "Melting-Point Diagrams of Refractory Oxides: IV Aluminum Oxide." *Z. anorg. u. allgem. Chem.*, 207, 1 (1932).
7. F. L. LaQue and H. R. Copson. "Corrosion Resistance of Metals and Alloys." 2nd Edition, Reinhold Publishing Corp., New York, p 628 (1963).
8. "Considerations on the Disposal of Radioactive Wastes from Nuclear Powered Ships into the Marine Environment," Publication 658, National Academy of Sciences, 1959.

HSH:bch