Contract No:

This document was prepared in conjunction with work accomplished under Contract No. DE-AC09-08SR22470 with the U.S. Department of Energy.

Disclaimer:

This work was prepared under an agreement with and funded by the U.S. Government. Neither the U.S. Government or its employees, nor any of its contractors, subcontractors or their employees, makes any express or implied:

1. warranty or assumes any legal liability for the accuracy, completeness, or for the use or results of such use of any information, product, or process disclosed; or 2. representation that such use or results of such use would not infringe privately owned rights; or 3. endorsement or recommendation of any specifically identified commercial product, process, or service. Any views and opinions of authors expressed in this work do not necessarily state or reflect those of the United States Government, or its contractors, or subcontractors.

Perspectives on DOE Consequence Inputs for Accident Analysis Applications

Kevin R. O'Kula*, David C. Thoman, Jonathan Lowrie, and Austin Keller[†]

Washington Safety Management Solutions LLC
P. O. Box 5388, Aiken, SC 29804-5388
*Email: kevin.okula@wsms.com; Phone: 803.502.9620
†Intern, Virginia Military Institute, Lexington, VA 24450

INTRODUCTION

Department of Energy (DOE) accident analysis for establishing the required control sets for nuclear facility safety applies a series of simplifying, reasonably conservative assumptions regarding inputs and methodologies for quantifying dose consequences. Most of the analytical practices are conservative, have a technical basis, and are based on regulatory precedent. However, others are judgmental and based on older understanding of phenomenology. The latter type of practices can be found in modeling hypothetical releases into the atmosphere and the subsequent exposure. Often the judgments applied are not based on current technical understanding but on work that has been superseded.

The objective of this paper is to review the technical basis for the major inputs and assumptions in the quantification of consequence estimates supporting DOE accident analysis, and to identify those that could be reassessed in light of current understanding of atmospheric dispersion and radiological exposure. Inputs and assumptions of interest include:

- Meteorological data basis
- Breathing rate
- Inhalation dose conversion factor.

A simple dose calculation is provided to show the relative difference achieved by improving the technical bases.

TECHNICAL BASIS REVIEW

A generalized, simplified version of the consequence terminology used in most DOE accident analysis is

Dose (Sv) = Source Term (Bq)

$$\gamma/Q (s/m^3) BR(m^3/s) IDCF (Sv/Bq)$$
 (1)

In Eq. 1, the source term represents the amount of respirable radiological material that is driven airborne to the environment by an imposed accident stress. The source term is often evaluated using the five-factor

formula, and is based on conservatively evaluating radiological inventories, accident types, facility information, and typically references DOE-HDBK-3010-94. [1] The remaining three terms are the primary areas of interest: (1) atmospheric transport dilution factor (χ/Q) based on meteorological conditions; (2) breathing rate (BR); and (3) inhalation dose conversion factor (IDCF). It should be noted that while Eq. 1 is broadly applicable to nonreactor nuclear facilities, it is best applied to noncriticality source terms where the dominant radiological hazard is due to radionuclides that are important through the inhalation pathway.

Table I is a listing of the three categories of input or parameter values and the corresponding regulatory or technical basis. The table summarizes key bases for DOE consequence analysis.

Table I. Major Input Requirements and Regulatory Basis

Table I. Major I	iiput Requirements	s and Regulatory Basis			
Input or	Value or	Regulatory/			
Parameter	Prescriptive	Technical Basis			
	Guidance				
A. Meteorological Basis					
1. statistical	95 th percentile	DOE-STD-3009-94,			
basis		App. A; Reg. Guide			
		1.145			
2.a stability	 ΔT 	Regulatory Guide			
categorization	• sigma-azimuth	1.23 (Draft)			
2.b stability	 sigma-azimuth 	EPA-454/R-99-005			
categorization	 solar radiation 				
	delta-T				
3.a surface	site evaluation	Rules of thumb in			
roughness	through surface	most consequence			
length	feature	software			
	characterization	applications			
3.b surface	$z_0=3$ cm;	DOE-STD-5506-			
roughness	~Prairie Grass	2007			
length	Experiments				
B. Breathing rate					
1. point value	Public/Worker:	DOE-STD-5506-			
	$3.33E-04 \text{ m}^3/\text{s}$	2007; DOE G 440.1-			
		3			

Input or	Value or	Regulatory/		
Parameter	Prescriptive	Technical Basis		
	Guidance			
2. range	Worker:3.33E-04	ICRP 66		
	$-4.69E-04 \text{ m}^3/\text{s}$			
C. Dose conversion factors				
1. Dose	Prescriptive	10 CFR 835		
terminology	dose evaluation			
	guidance			
2. Weighting	-	ICRP 60		
factors;				
biokinetics				
model				
3. Respiratory	-	ICRP 66		
tract model				
4. Particle size	Physicochemica	ICRP 68;		
distribution*	1 grouping	ICRP 71		
5. DCFs	Worker	ICRP 30; ICRP 68;		
	DCFs; Public	ICRP 72; also		
	DCFs	DOE-STD-5506-		
		2007 and DOE-		
		STD-1189-2008		
*The particle size also establishes the deposition				
velocity, which affects quantification of χ/Q .				

Meteorological Basis

The meteorological basis for the consequence calculation is the 95th percentile, direction-independent level of consequence (γ/Q) is summarized in Appendix A to DOE-STD-3009-94.[2] Data for forming the basis for the 95th percentile is often processed using Regulatory Guide 1.23 – including the technique for determining the stability category for the set of meteorological data. [3] Depending on site surface characteristics and meteorological data collection instrumentation, the methodology can be inaccurate in assigning data to stable categories. More accurate techniques are provided by the EPA. [4] A second factor that can be identified is the surface roughness length (z_0) , a measure of the mechanical turbulence introduced by surface features. A recent DOE Standard specifies $z_0 = 3$ cm for all facility dose evaluations, characteristic of dispersion tests over relatively flat, treeless terrain. [5]

An area of uncertainty in the meteorological phase of the analysis is the physicochemical characteristics of the radiological material during dispersion. Current, working assumptions can be generalized as follows: (1) the radiological material in transport is the same form as what existed pre-accident; and (2) particle size and related deposition characteristics are based on a monosize assumption. A deposition velocity of 1 cm/s is typically specified based on a particle size of 2-4 microns AED.

Breathing Rate

The breathing rate is a point value in consequence analysis and is tied to respiratory considerations. For recommendations for both worker and public receptors, the most authoritative guidance is ICRP 66. [6] The recommendation for the breathing rate in DOE-STD-5506-2007 is consistent with the ICRP guidance, and specifies the same value as derived in DOE G 440.1-3. [7] A point of departure is that a range of light to heavy activity breathing rate is recommended for the worker in Ref. 6, rather than a single breathing rate. A range of values is also recognized for the public receptor and is based on age, gender, and activity level. [6]

Dose Conversion Factors

With issuance of 10 CFR 835, DOE radiological assessments are to implement the newer biokinetics model and weighting factors, as well as the updated respiratory model. [8, 9] The radiological dose conversion factors that complement the improved models are contained in ICRP 68 and ICRP 72 for worker and general public receptors. [10, 11] This information reflects our current state of knowledge and is an advance over previous IDCFs. [12] An area of uncertainty is the particle size, and the default value is generally applied (1 μm or 5 μm activity median aerodynamic diameter (AMAD)) for the inhalation pathway for unmitigated analysis. [13]

Upon review of the three categories of input specification and guidance, the meteorological aspect is technically less accurate. The factors in this category that could be technically improved are stability category basis and the surface roughness length. The IDCF data upgrade is already reflected in current-day standards.

RESULTS

To better estimate the overall effects of the current updated inputs and assumptions with those less accurate, a sample calculation is performed using the MACCS2 code for a postulated Savannah River Site release. [14] The baseline case is a ground-level three-minute release of one curie of ²³⁹Pu and the dispersion analysis is performed with $z_0 = 3$ cm. The meteorological data set is prepared using a Ref. 3 approach for analyzing stability categories. The analysis is repeated with a more accurate (Ref. 4) method of binning stability categories. A dose comparison is shown with the percentage change as a function of distance in Table II. The second half of the table repeats both cases with $z_o = 100$ cm (forest surface cover). A third identified input, deposition velocity, is constant at 1 cm/s. The dose estimates shown in Table II provide a sense of the dose change with more accurate models and assumptions for stability class and surface roughness length.

Table II. Comparison of 95th Percentile Doses for Data Processed with Two Methods of Stability Categorization and Two Surface Roughness Lengths (3 cm and 100 cm)

	95 th	95 th			
Distance	Percentile	Percentile	Difference		
(km)	Dose (Sv),	Dose (Sv),	(%)		
	(1997-2001)	(2002-2006)			
$z_o = 3$ cm					
0.1	2.17E+00	1.75E+00	-19.4		
0.6	1.59E-01	1.21E-01	-23.9		
1.7	2.44E-02	2.06E-02	-15.6		
5.2	4.12E-03	3.32E-03	-19.4		
9.2	1.36E-03	1.08E-03	-20.6		
11.5	9.08E-04	7.37E-04	-18.8		
z _o = 100 cm					
0.1	1.23E+00	9.51E-01	-22.7		
0.6	8.65E-02	7.26E-02	-16.1		
1.7	1.58E-02	1.20E-02	-24.1		
5.2	2.55E-03	2.15E-03	-15.7		
9.2	9.47E-04	7.82E-04	-17.4		
11.5	6.84E-04	5.42E-04	-20.8		

CONCLUSIONS

A review of the three primary categories of inputs to a generalized consequence analysis for DOE safety analysis has been performed. The least reflective of current, state-of-the-art understanding of radiological atmospheric release and exposure from accident conditions are those in the meteorological area. Key uncertainties include particle deposition velocity and the particle size distribution associated with inhalation pathway.

REFERENCES

- 1. U.S. DEPARTMENT OF ENERGY, Airborne Release Fractions/Rates and Respirable Fractions for Nonreactor Nuclear Facilities, DOE-HDBK-3010-94, Change Notice 2, Washington, DC, (March 2000).
- 2. U.S. DEPARTMENT OF ENERGY, Preparation Guide for U.S. Department of Energy Nonreactor Nuclear Facility Safety Analysis Reports, DOE-STD-3009-94, Washington, DC, Change Notice 3 (March 2006).

- 3. U.S. NUCLEAR REGULATORY COMMISSION, *Meteorological Programs for Nuclear Power Plants*, Regulatory Guide 1.23, Washington, DC (1980 Rev. 1 Draft).
- U.S. ENVIRONMENTAL PROTECTION AGENCY, Meteorological Monitoring Programs for Regulatory Modeling Applications, EPA-454/R-99-005, Washington, DC (2000).
- 5. U.S. DEPARTMENT OF ENERGY, *Preparation of Safety Basis Documents for Transuranic Waste Facilities*, DOE-STD-5506-2007, Washington, DC, (April 2007).
- 6. INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, *Human Respiratory Tract Model for Radiological Protection*, ICRP Publication 66 *Annals of the ICRP* Volume 24, Nos. 1-3 (1994).
- 7. U.S. DEPARTMENT OF ENERGY, Implementation Guide for use with DOE Order 440.1, Occupational Exposure Assessment, DOE G 440.1-3, Washington, DC, (3-30-98).
- 8. 10 CFR 835.
- 9. INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Recommendations of the International Commission on Radiological Protection, ICRP Publication 60 Annals of the ICRP Volume 21, Nos. 1-3 (1991).
- 10. INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Dose Coefficients for Intakes of Radionuclides by Workers: A Report of a Task Group of Committee 2 of the International Commission on Radiological Protection Replacement of ICRP Publication 61, ICRP Publication 68 Annals of the ICRP Volume 24, No. 2 (1994).
- 11. INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Age-dependent Doses to the Members of the Public from Intake of Radionuclides: Part 5, ICRP Publication 72 Annals of the ICRP Volume 26, No. 1 (1996).
- 12. INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, *Limits for Intakes of Radionuclides by Workers; Annals of the ICRP Volume 19, No 4*, ICRP Publication 30, (1979).
- 13. INTERNATIONAL COMMISSION ON RADIOLOGICAL PROTECTION, Age-dependent Doses to the Members of the Public from Intake of Radionuclides: Part 4 Inhalation Dose Coefficients, ICRP Publication 71, Annals of the ICRP, Volume 25, No. 3, (1996).
- D.I. CHANIN, ET AL., Code Manual for MACCS2: Volume 1, User's Guide; NUREG/CR-6613 (SAND97-0594), Sandia National Laboratories, published by the U.S. Nuclear Regulatory Commission, Washington, DC, 1998.