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AEC RESEARCH AND DEVELOPMENT REPORT

# A MECHANISM FOR STRESS CORROSION CRACKING OF STAINLESS STEEL IN REACTOR SYSTEMS

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A MECHANISM FOR STRESS CORROSION CRACKING  
OF STAINLESS STEEL IN REACTOR SYSTEMS

by

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#### ABSTRACT

*was developed that could explain the stress corrosion cracking*

A mechanism for chloride stress corrosion cracking of Type 304 stainless steel that has been sensitized, pickled, and dry ground, was shown to involve the chloride-bearing alumina present in the moderator-coolant system of Savannah River reactors. Chloride is absorbed by the alumina during its formation on hot fuel cladding; iron, an impurity in the cladding, greatly enhances chloride absorption. Alumina spalls and is transported by the coolant to the stainless steel surfaces. Chloride is released from the alumina by the acidic reaction at anodic sites on the steel. The negative chloride ion is attracted to the positive anodic site and initiates stress corrosion cracking.

Cracking of susceptible stainless steel by this mechanism can be reduced by:

- Controlling coolant pH (pD) between 5.0 and 5.2 to prevent general release of chloride.
- Avoiding addition of chlorides and chlorinated hydrocarbons to the reactor.
- Blocking anodic sites by inhibitors or by changing the potential of the stainless steel.

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## INTRODUCTION

Several stress corrosion cracks have occurred in the Type 304 stainless steel cooling systems of Savannah River reactors: the first one after five years operation and subsequent ones at irregular intervals. Inspection of the cracks revealed intergranular corrosion confined to areas severely sensitized\* by hot-forming and pickling operations during reactor construction.<sup>1-3</sup> Cracking was unexpected because the heavy water used as coolant and neutron moderator is very pure; conductivity is typically 3 micromhos/cm. The ionic content (0.5 ppm  $\text{NO}_3^-$ , 0.04 ppm  $\text{SO}_4^{2-}$ , 0.02 ppm  $\text{Cl}^-$ ) is generally considered noncorrosive to Type 304 stainless steel, even in the sensitized condition.<sup>1</sup>

Prevention of cracking is important to maintain continuity of reactor operation, eliminate expensive repairs, and minimize heavy water losses.

Chloride-bearing alumina from corroded aluminum reactor components has been implicated<sup>1</sup> in the cracking process. The cracking was observed to have both trans- and intergranular characteristics, but the mechanism was not known. The work reported here was designed to investigate the mechanisms by which the chloride-bearing alumina might cause stress corrosion-cracking.

Oxygen increases the sensitivity of chloride stress corrosion cracking of stainless steel or lowers the chloride content necessary for cracking.<sup>4,5</sup> Oxygen also contributes to intergranular stress corrosion cracking of corrosion resistant steel above about  $\sim 250^\circ\text{C}$  in water at pH 10 with low oxygen content ( $\sim 0.1$  ppm) or in neutral water with a high oxygen content (100 ppm).<sup>6</sup> These effects have been reviewed critically,<sup>5</sup> but were not evaluated in this study.

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\* "Sensitized" is defined as heated in the range of  $540-870^\circ\text{C}$ , which precipitates chromium carbides in grain boundaries of the metal. In this condition, the material is very susceptible to intergranular corrosion.

## SUMMARY

Three types of crystalline alumina were identified in the reactor system:  $\alpha\text{Al}_2\text{O}_3 \cdot \text{D}_2\text{O}$  (Böhmite),  $\alpha\text{Al}_2\text{O}_3 \cdot 3\text{D}_2\text{O}$  (Gibbsite), and  $\beta\text{Al}_2\text{O}_3 \cdot 3\text{D}_2\text{O}$  (Bayerite). The  $\alpha$  monohydrate and  $\beta$  trihydrate, which form on the aluminum-clad fuel elements and aluminum housings, spall and become suspended in the moderator as  $\beta$  trihydrate through stresses induced by the conversion from  $\alpha$  monohydrate to  $\beta$  trihydrate and from thermal shock due to reactor shutdown and startup. The very finely divided turbidity is transported by the moderator and electrophoretically deposited on the stainless steel components where conversion to  $\alpha$  trihydrate occurs. Turbidity sometimes contains high concentrations of  $\alpha$  trihydrate due to erosion of deposits on the stainless steel by hydraulic upset of the cooling system.

Alumina absorbs chloride during oxide growth on aluminum surfaces. Very little chloride is absorbed on crystalline alumina. Iron present as an impurity in the aluminum is the primary contributor to chloride absorption during corrosion film growth. Silicon, also present as an impurity in the aluminum, is the primary contributor to rapid growth of the corrosion film.

A change in pH is the only effective mechanism for releasing absorbed chloride from alumina under typical reactor operating conditions. Stainless steel cracking observed in sensitized, pickled, and ground areas is indicated to be due to deposition of chloride-bearing alumina at anodic areas that are typically acidic. The negatively charged chloride is released and attracted to these positively charged areas to initiate the stress corrosion cracking sequence.

Stainless steel cracking that is caused by chloride-bearing alumina can be reduced by several methods. Maintaining the coolant pH (pD) between 5.0 and 5.2 reduces the general release of chlorides. Every effort should be made to prevent accidental addition of chloride to the reactor. Chlorinated hydrocarbons, which typically chemisorb on alumina, should not be used for degreasing when other reagents will serve equally well. The anodic sites where the cracks initiate might be blocked through the use of inhibitors or be inactivated by changing the stainless steel potential.

## DISCUSSION

### DESCRIPTION OF REACTOR COOLING SYSTEM

The nuclear reactors at Savannah River are constructed primarily of as-welded 304 stainless steel, small portions of which were severely sensitized by hot-forming operations during fabrication. The reactor vessel is a large cylindrical tank that contains the aluminum-clad nuclear fuel elements. Heavy water, the coolant and neutron moderator, circulates through the reactor vessel and fuel elements, and is discharged through six nozzles symmetrically spaced around the bottom of the tank. Large pumps circulate the heavy water through heat exchangers and back to a plenum in the top of the reactor vessel. Several side streams are taken from the main heavy water stream. One of these flows through an ion exchange resin bed for purification. A simplified schematic of the system is shown in Figure 1.

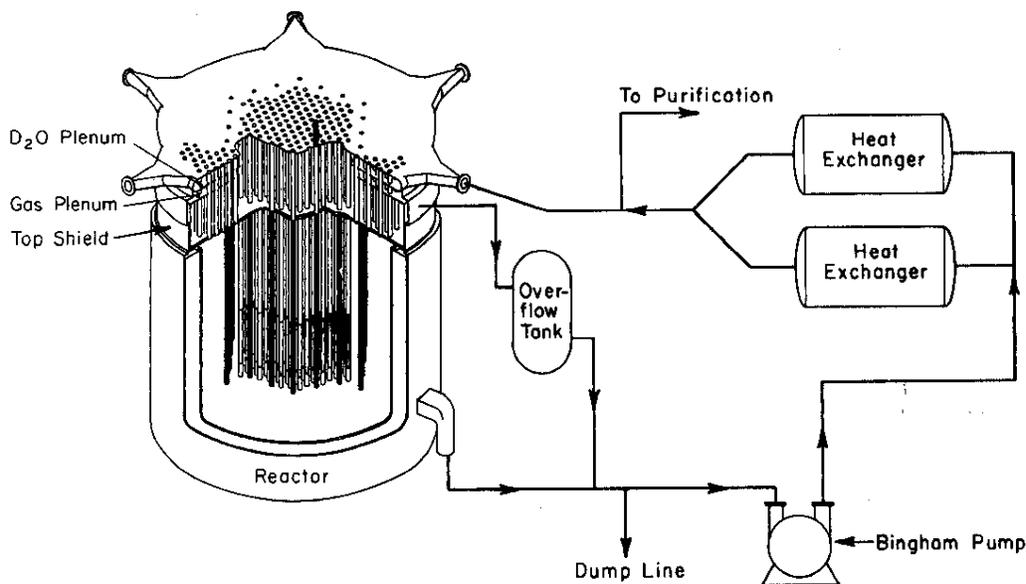


FIG. 1 SIMPLIFIED SCHEMATIC OF SRP NUCLEAR REACTOR

Mixed inter- and transgranular cracking, a variety of chloride stress corrosion, has occurred in several tank outlet nozzles and pump discharge lines. Relatively few cracks have occurred; the first one after more than five years of operation and subsequent ones at irregular intervals.

Films of alumina, formed by corrosion of aluminum-clad fuel elements, are deposited in portions of the reactor system. Analyses of the alumina deposits show chloride concentrations ranging from 10-10,000 ppm (0.001-1 wt %).

## CHARACTERIZATION OF REACTOR DEPOSITS

Specimens of alumina were collected from the reactor system at several different locations. Scrapings were obtained from the 1) pump impellers and housings, 2) heat exchangers, 3) fuel cladding, and 4) various portions of piping. Moderator turbidity composed of finely divided alumina was trapped in the ion exchange resin bed and was also examined. Table I shows the various types of alumina found in the reactor system.

TABLE I

### Aluminas Formed in Savannah River Reactor Systems

Formula	Structure, Crystal Class	Common Name	Principal Location
$\alpha\text{Al}_2\text{O}_3 \cdot \text{D}_2\text{O}$	$\text{AlO}(\text{OD})$ - orthorhombic	Böhmite	Fuel cladding
$\alpha\text{Al}_2\text{O}_3 \cdot 3\text{D}_2\text{O}$	$\text{Al}(\text{OD})_3$ - monoclinic	Gibbsite	Steel surfaces
$\beta\text{Al}_2\text{O}_3 \cdot 3\text{D}_2\text{O}$	$\text{Al}(\text{OD})_3$ - monoclinic	Bayerite	Moderator turbidity

The major quantity of alumina within the reactor at any given time is the  $\alpha\text{Al}_2\text{O}_3 \cdot 3\text{D}_2\text{O}$  deposited on the stainless steel components. Typical analyses of deposits removed from the reactor tank outlet are shown in Table II. Typical deposits are tan, not white as would be expected for pure alumina.

TABLE II

### Analyses of Alumina Adhering to Stainless Steel

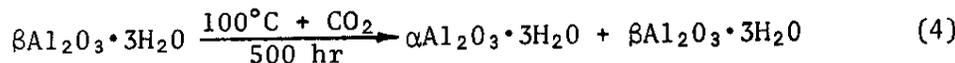
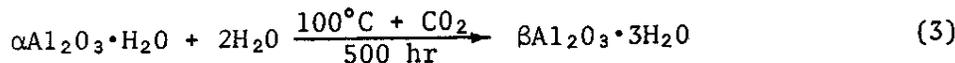
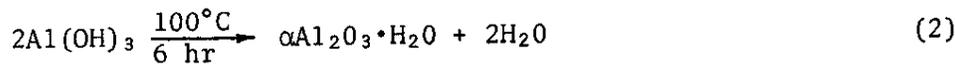
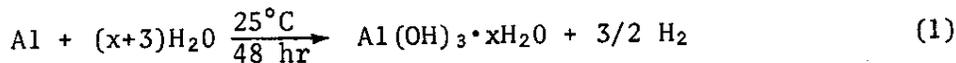
Analysis Method	Determination	Amount
X-ray diffraction	$\alpha\text{Al}_2\text{O}_3 \cdot 3\text{D}_2\text{O}$	Major
Gamma pulse height	$^{95}\text{Zr}$ - $^{95}\text{Nb}$	Major
	$^{51}\text{Cr}$	Major
	$^{60}\text{Co}$	Minor
Neutron activation	C1	10-10,000 ppm
Emission spectroscopy	Al	>10 wt %
	Fe	5
	Cr	2
	Mn	1
	Ni	1
	Si	1
	Mg	0.5
	Ba, Ca, Cu, Sn, Ti	≤0.1

Although the major component of the scrapings is alumina, many impurities were identified by emission spectroscopy. Part of the iron, nickel, and silicon is from aluminum alloys, but most is probably from steel corrosion. The chromium and manganese are from steel corrosion; the magnesium is from aluminum corrosion.

#### ALUMINA FORMATION, TRANSPORT, AND DEPOSITION

The consistent identification of  $\alpha\text{Al}_2\text{O}_3 \cdot 3\text{D}_2\text{O}$  as the major constituent in alumina deposits on stainless steel initially appeared to be anomalous. The  $\alpha$  trihydrate is normally synthesized by aging hydrated aluminum oxide in solutions at pH 10 or by precipitating  $\text{AlO}_2^-$  at pH 9 or greater. The coolant is held at pH 5; therefore, the direct formation of  $\alpha$  trihydrate is unlikely because of the small quantity of  $\text{AlO}_2^-$  ion and the large quantity of  $\text{Al}^{3+}$  ( $\text{Al}^{3+}/\text{AlO}_2^- = 100/1$ ). The anticipated products at pH 5 would be  $\beta\text{Al}_2\text{O}_3 \cdot 3\text{D}_2\text{O}$  below  $80^\circ\text{C}$  and  $\alpha\text{Al}_2\text{O}_3 \cdot \text{D}_2\text{O}$  above  $80^\circ\text{C}$ , due to crystallization of amorphous  $\text{Al}(\text{OD})_3 \cdot x\text{D}_2\text{O}$ .

After several attempts, the anomalous  $\alpha$  trihydrate was synthesized by an indirect route; similar reactions would be expected during fabrication and irradiation of aluminum-clad nuclear fuels. Type 1100 aluminum was amalgamated to make it susceptible to corrosion at  $25^\circ\text{C}$  in water. The water was adjusted to pH 5 and readjusted daily throughout the experimental period during which the effects of increasing the temperature and of the addition of  $\text{CO}_2$  were explored. The following transitions were observed by X-ray diffraction.



Reaction 1 can occur in moist air and in water below 80°C. Reaction 2 occurs above 80°C. Reaction 3 could occur during reactor operation. Carbon dioxide was added during Reaction 3 because this gas had been used during reactor startup to maintain moderator pH (pD). Reaction 4, the transition from the  $\beta$  to  $\alpha$  trihydrate, is not expected to occur rapidly. Calculated  $\Delta F^\circ$  values are -552 kcal for the formation of the  $\beta$  trihydrate and -555 kcal for  $\alpha$  trihydrate.<sup>7</sup> These values indicate that Reaction 4 is slow even at elevated temperature. In fact the reaction was not observed during 10 years in a sealed sample in pure water at 25°C.<sup>8</sup>

The temperature and pH and the presence of CO<sub>2</sub> probably account for the formation of the  $\alpha$  trihydrate in a relatively short time in these tests. The tests were designed to simulate reactor conditions with the exception of the neutron flux.

Some of the solid material that causes turbidity in the moderator was separated from ion exchange resin and was shown to be mainly  $\beta\text{Al}_2\text{O}_3 \cdot 3\text{D}_2\text{O}$  containing 360 ppm Cl (Figure 2).

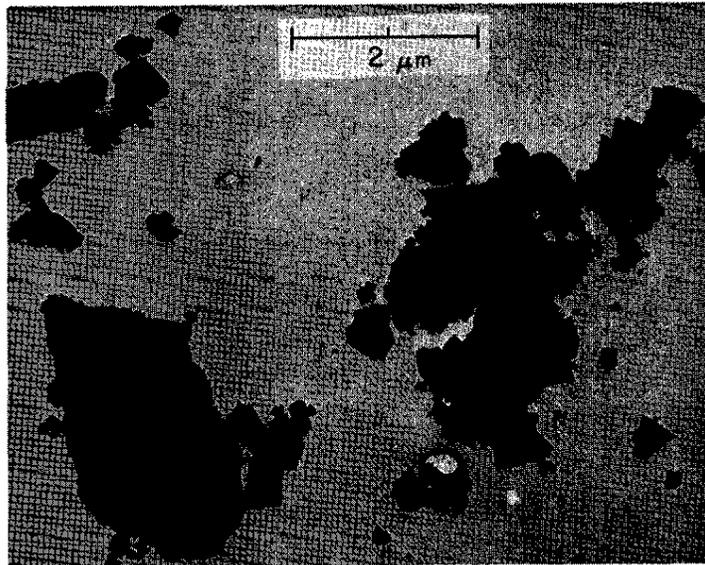


FIG. 2 TURBIDITY FROM ION EXCHANGE COLUMN

From the observed crystalline forms in the reactor system and the synthesis of  $\beta\text{Al}_2\text{O}_3 \cdot 3\text{H}_2\text{O}$  in the laboratory, the deposition of alumina on stainless steel most logically follows the pattern:

1. Aluminum corrodes at low temperature to form  $\text{Al}(\text{OH})_3 \cdot x\text{D}_2\text{O}$  which converts to  $\alpha\text{Al}_2\text{O}_3 \cdot \text{D}_2\text{O}$  during reactor startup or forms  $\alpha\text{Al}_2\text{O}_3 \cdot \text{D}_2\text{O}$  directly during reactor operation.
2.  $\alpha\text{Al}_2\text{O}_3 \cdot \text{D}_2\text{O}$  on the fuel hydrates to  $\beta\text{Al}_2\text{O}_3 \cdot 3\text{D}_2\text{O}$  and then spalls because of stresses due to re-crystallization from orthorhombic to monoclinic.
3.  $\beta\text{Al}_2\text{O}_3 \cdot 3\text{D}_2\text{O}$ , the main oxide present in turbidity as minute particles or agglomerates, is transported throughout the system.
4. The minute particles ( $<1 \mu\text{m}$ , Figure 2) electro-phoretically deposit on the steel and gradually transform from  $\beta$  to  $\alpha\text{Al}_2\text{O}_3 \cdot 3\text{D}_2\text{O}$ .

Step 2 assumes that thermal shock has not occurred to remove protective aluminum oxide. Thermal shock sometimes occurs in reactor operation and accounts for the mixture of  $\alpha$  monohydrate and  $\beta$  trihydrate found in turbidity. The presence of  $\alpha$  trihydrate in turbidity is due to removal of the electrophoretic deposit by erosion or hydraulic upset.

## CHLORIDE ABSORPTION BY ALUMINA

Alumina from Pure Aluminum and Commercial Alloys

Chloride was not absorbed by well-crystallized alumina suspended in water. As described in the Appendix, a suspension of amorphous  $\text{Al}(\text{OH})_3 \cdot x\text{H}_2\text{O}$  was produced by immersing an amalgamated aluminum surface in water at  $25^\circ\text{C}$ . After 8 days at pH 5, a well-crystallized  $\beta\text{Al}_2\text{O}_3 \cdot 3\text{H}_2\text{O}$  had formed in the solution. At  $100^\circ\text{C}$ , amorphous  $\text{Al}(\text{OH})_3 \cdot x\text{H}_2\text{O}$  was again the initial product, but after about 4 hours the crystalline form was  $\alpha\text{Al}_2\text{O}_3 \cdot \text{H}_2\text{O}$ . Little chloride adsorption was observed either at 25 or  $100^\circ\text{C}$  when these crystallized products were added to pH 5 solutions containing  $^{36}\text{Cl}^-$  as part of 1.5 ppm  $\text{Cl}^-$ . After 17 days the experiment was discontinued.

On the other hand chloride was absorbed on alumina as it was formed at a corroding surface. The aluminum was not amalgamated in this case, and a decrease in the radioactivity of the solution compared to a blank solution indicated adsorption of chloride. After 17 days a direct counting of the aluminum surface verified that the chloride had actually been absorbed in the oxide.

Table III compares the quantity of chloride absorbed by both particulate alumina and aluminum surfaces. These data suggest that 1) the absorption of chloride is high on oxidizing metal surfaces, 2) absorption occurs during the initial stages of film formation, and 3) absorption is not due to an alloying agent (Ni) but to an impurity in the aluminum. Deposition of chloride ions on selective locations, believed to be impurity sites, have been reported to occur in anodizing solutions.<sup>9</sup> Table III also shows that particulate alumina is a very poor absorber of chloride compared to a growing film of alumina. Even though the molar ratio of Al/Cl was 1000:1 in the particulate suspensions, little chloride was absorbed at 25 or 100°C. Impure alumina from 1100 alloy absorbed more chloride than pure alumina, but even these concentrations of chloride in alumina were low compared to concentrations found in the reactor system.

TABLE III  
Absorption of Chloride by Alumina

Material <sup>a</sup>	Temp, °C	Absorbed Chloride, ppm
Particulate Alumina		
$\alpha$ Al <sub>2</sub> O <sub>3</sub> ·H <sub>2</sub> O - High purity aluminum	100	<0.2 <sup>b</sup>
$\beta$ Al <sub>2</sub> O <sub>3</sub> ·3H <sub>2</sub> O - High purity aluminum	25	<0.2 <sup>b</sup>
$\alpha$ Al <sub>2</sub> O <sub>3</sub> ·H <sub>2</sub> O - 1100 aluminum	100	1
Corrosion Film on Aluminum Surfaces		
High purity aluminum	100	16
1100 aluminum	100	100
X8001 aluminum	100	100

<sup>a</sup> High purity: 99.9995+% Al  
 1100:  $\geq$ 99.0% Al, <1.0% Si + Fe, <0.2% Cu, <0.1% Zn, <0.5% Mn  
 8001: 1100 Al + 0.9 to 1.1% Ni  
<sup>b</sup> Lower limit of detection.

To verify that nickel was not the cause of chloride sorption, two specimens (one of 1100 Al and one of 8001) were oxidized in water for 10 days. The pH was held at 5, the temperature at 100°C, and the concentration at 1.5 ppm of Cl<sup>-</sup> containing <sup>36</sup>Cl<sup>-</sup>. The oxide was 1.2 μm thick and had absorbed 6-8 x 10<sup>-8</sup> g Cl<sup>-</sup>/cm<sup>2</sup> in both cases. Autoradiographs of the X8001 alloy showed that chloride absorption was generally uniform and could not be ascribed to the relatively large particles of the nickel-rich phase (Figure 3).

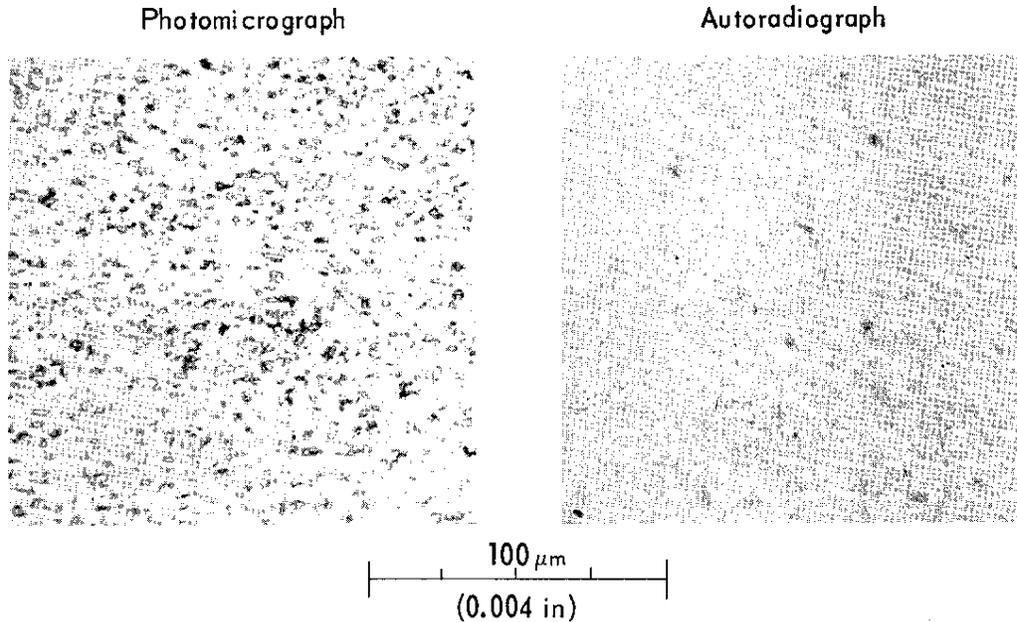


FIG. 3 NICKEL DISTRIBUTION IN 8001 ALUMINUM VS CHLORIDE ABSORPTION IN OXIDE FILM  
Note that the chloride in the autoradiograph (dark dots) does not correspond to the nickel distribution in the alloy. Nickel-rich phases do not absorb chloride preferentially.

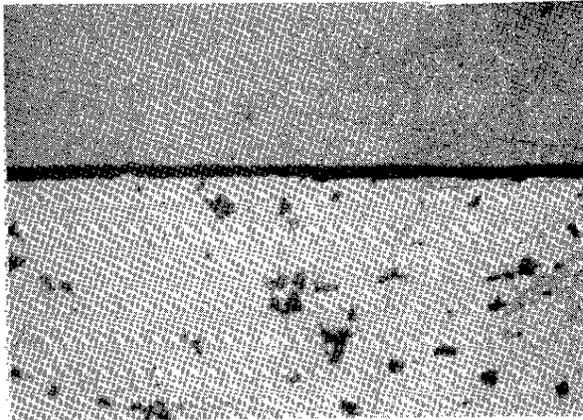
## Alumina Formed on Binary Alloys

To study the effect of impurities on the absorption of chloride by aluminum, special binary alloys were prepared by vacuum melting. These alloys contained 99.9995+% aluminum plus one of the reagent-grade metals Fe, Si, or Cu, each within the concentration specifications of 1100 and 8001 alloys. After annealing, the alloys were corroded for 335 hours at 100°C in deionized water containing 1.5 ppm Cl<sup>-</sup> with <sup>36</sup>Cl<sup>-</sup> tracer. Throughout the test, pH was maintained at 5. The specimens were counted for <sup>36</sup>Cl<sup>-</sup> activity, and the thicknesses of the films were measured optically (Figure 4). This figure shows that small concentrations of alloying elements can have large effects on aluminum corrosion. A summary of the absorption of chloride or the binary alloys is shown in Table IV.

TABLE IV  
Chloride Absorption by Aluminum Corrosion  
Film on Binary Alloys

Wt % Alloying Element	Film Thickness, μm	Chloride in Film	
		μg/cm <sup>2</sup>	ppm
0.51 Fe	2	0.13	270
0.22 Cu	9	0.054	25
0.21 Si	27	0.078	12

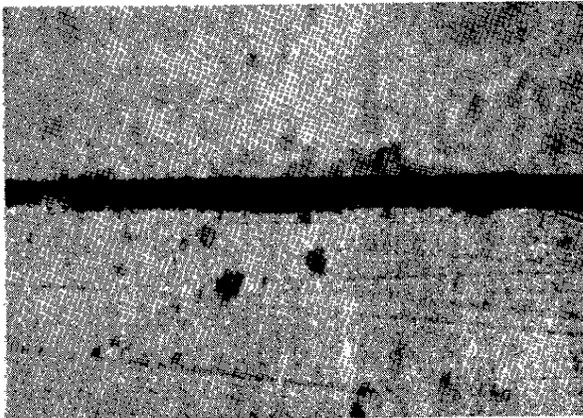
Iron was the most effective impurity of the three in increasing the absorption of chloride. Note that under conditions similar to those in the reactor, silicon in the tenths-weight-percent range causes very rapid corrosion. This phenomenon has been observed by others and has led to recommendation of very low<sup>10,11</sup> or very high<sup>12</sup> silicon concentrations for aluminum alloy used as cladding for nuclear fuels. These experiments indicate that an aluminum-iron alloy would be good for fuel cladding because of the thin film formed.



Edge support

Corrosion film (2  $\mu\text{m}$  avg.)

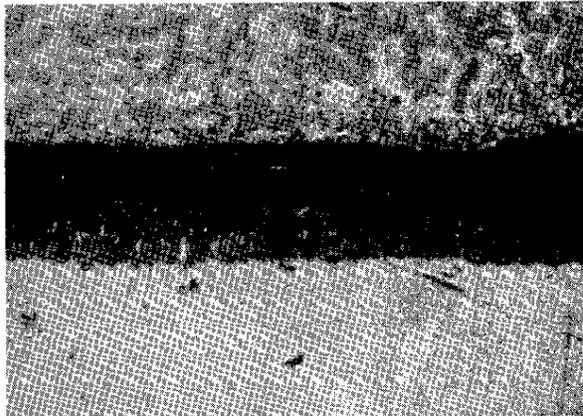
Al-0.51% Fe



Edge support

Corrosion film (9  $\mu\text{m}$  avg.)

Al-0.22% Cu



Edge support

Corrosion film (27  $\mu\text{m}$  avg.)

Al-0.21% Si

20  $\mu\text{m}$

**FIG. 4 CORROSION OF BINARY ALUMINUM ALLOYS**  
Corroded at 100°C for 335 hrs in pH 5 deionized water containing 1.5 ppm Cl<sup>-</sup>.  
Note the large differences in corrosion due to small quantities of alloying agents.

## DESORPTION OF CHLORIDE FROM ALUMINA

The intermittent nature of stainless steel cracking suggests that chloride release from alumina is not continuous, and may be triggered by some variable in the coolant. Therefore, the effects of  $H^+$ ,  $CO_2$ ,  $H_2O_2$ ,  $NO_3^-$ , and  $SO_4^{2-}$ , known to occur at varying concentrations in the coolant, were investigated.

A mixture of  $\alpha$  and  $\beta Al_2O_3 \cdot 3H_2O$  that contained 50 ppm of  $Cl^-$  including  $4.9 \times 10^3$  counts of  $^{36}Cl^-/(min)(g)$  was prepared from 1100 aluminum alloy. Portions of this active alumina were added to deionized water containing one of the characteristic moderator constituents. To determine the effect of  $H^+$ , the pH was varied from 4 to 7; in all other cases, pH was held constant at 5. The temperature of the alumina suspension was held at  $100^\circ C$  for 200 hours, and the activity of the aqueous phase was determined periodically.

The release of chloride to the aqueous phase with changes in pH is shown in Figure 5. The smallest amount of chloride is released at pH 5. Although larger amounts are released at both higher and lower pH, pH changes in the acid direction release more chloride than equivalent changes in the less acid direction.

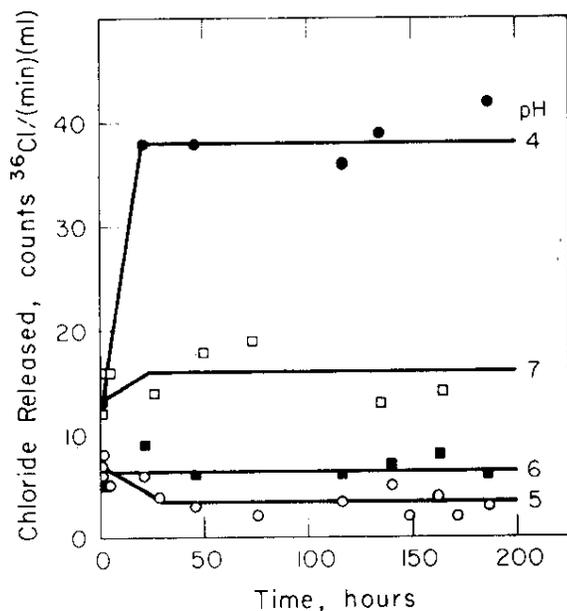


FIG. 5 EFFECT OF pH ON CHLORIDE RELEASE FROM ALUMINA

Comparison of the release of chloride to the solubility of  $\alpha$  and  $\beta\text{Al}_2\text{O}_3 \cdot 3\text{H}_2\text{O}^4$  showed a direct correlation as illustrated in Figure 6 and Table V. Figure 6 shows that the chloride release shown in Figure 5 follows the same pattern as solubility.<sup>7</sup> Table V gives a quantitative comparison between Figures 5 and 6.

TABLE V

Chloride Release from 1100 Aluminum Corrosion Product Compared to Alumina Solubility

pH	Relative Cl <sup>-</sup> Released from Corrosion Product	Relative Solubility of $\alpha$ and $\beta\text{Al}_2\text{O}_3 \cdot 3\text{H}_2\text{O}^4$
4	12.3	800
5	1.0	1
6	2.3	4
7	5.3	40

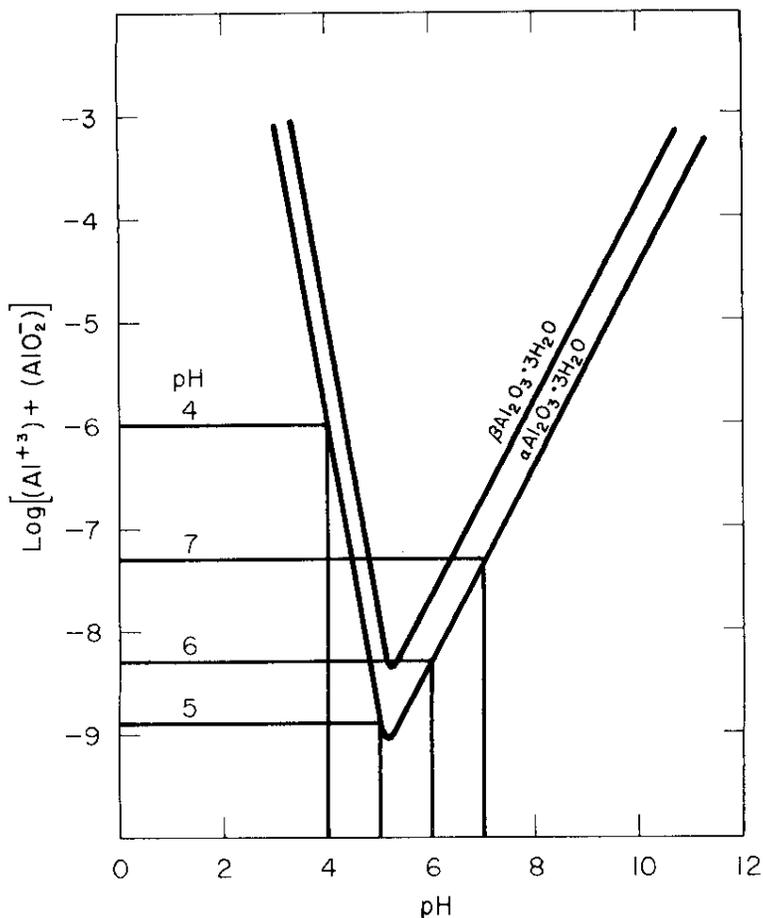


FIG. 6 SOLUBILITIES OF ALUMINA AT 25°C(2)

The pH effects on release of chloride from corrosion product are of smaller magnitude than the pH effects on the solubility of pure alumina. This distinction is attributed to the effect of iron in either preventing the release of chloride during dissolution of the alumina or decreasing the solubility of the corrosion product below that of pure alumina.

The addition of  $\text{NO}_3^-$ ,  $\text{CO}_2$ , and  $\text{H}_2\text{O}_2$  to pH 5 deionized water at  $100^\circ\text{C}$  containing a suspension of alumina did not affect the desorption of chloride. Table VI shows the concentrations of ions and compounds used in the experiment compared to typical moderator concentrations.

TABLE VI  
Solute Concentrations for Determining  
Chloride Release, ppm

<u>Solute</u>	<u>Experimental Solutions</u>	<u>Moderator</u>
$\text{CO}_2$	50-200	15
$\text{H}_2\text{O}_2$	5-100	2
$\text{NO}_3^-$	1	0.001-0.5
$\text{SO}_4^{2-}$	0.5	0.02 -0.4

The results in the above experiment were identical to the results at pH 5 without addition of the solutes.

## MECHANISM OF STAINLESS STEEL CRACKING

To test the effects of alumina containing absorbed chloride specimens of 304 stainless steel were prepared by techniques simulating those experienced by metal in the cracked portion of the reactor. The steel was sensitized, pickled with 1M HF - 4M HNO<sub>3</sub>, dry ground, and stressed. Alumina (3Al<sub>2</sub>O<sub>3</sub>·3H<sub>2</sub>O) containing 380 ppm Cl<sup>-</sup> was added to pH 5 deionized water to form a suspension. After 4 hours at 25°C, the aqueous phase in equilibrium with the alumina was found to contain 0.3 ± 0.1 ppm Cl<sup>-</sup>. The sample was immersed, the pH was increased to 7, and exposure was continued for up to 36 days at 90 to 95°C. Figure 7 shows one of two pits that occurred on one specimen at about 75% of the maximum stress. This

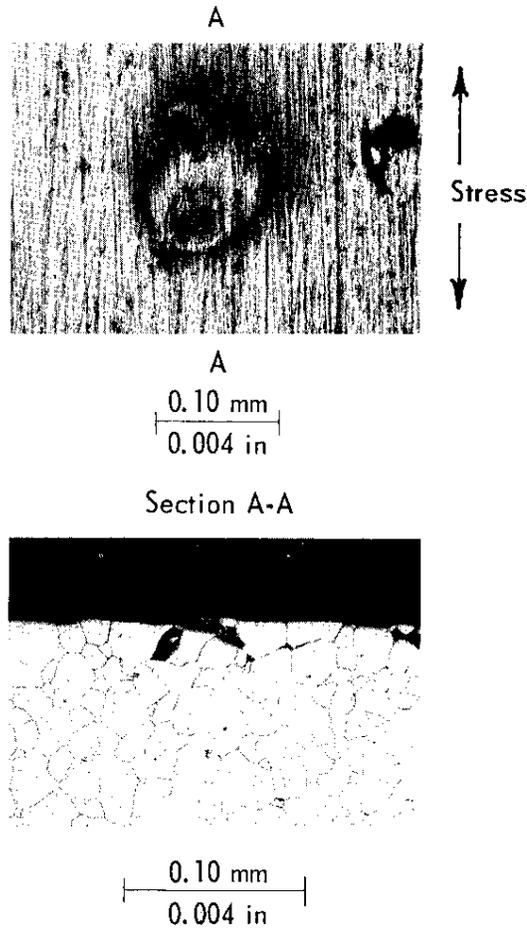


FIG. 7 PITTING OF 304 STAINLESS STEEL BY CHLORIDE-BEARING ALUMINA

Pitting due to anodic reaction. Note intergranular attack sufficient to loosen entire grains, and possible intergranular crack initiation. Grain loosening is a characteristic of the cracked areas found in the reactor system.

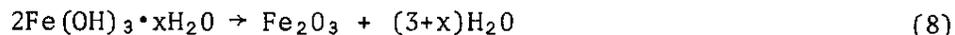
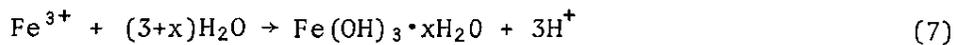
specimen did not crack after 240 hours exposure; a metallographic section revealed intergranular penetration to the extent that whole grains dropped from the sample, however Figure 8 shows another specimen that did crack in two locations under maximum stress. Cross sections of these cracks were typical of stress corrosion cracking.

Analysis of the solution after the 240-hour test showed  $1.1 \pm 0.1$  ppm  $\text{Cl}^-$ , equivalent to a release of 25% of the chloride that had been absorbed on the alumina. If all the chloride had been released at once, the chloride ion concentration would not have exceeded 4 ppm.

Cracked areas in the reactor system are normally coated with alumina and stained with ferric oxide ( $\text{Fe}_2\text{O}_3$ ) in the cracked area. The ferric oxide shows that oxidation is taking place, and the area is therefore anodic. Oxidation of steel in an aqueous system containing oxygen occurs by Reactions 5 and 6.



Because  $\text{Fe}^{3+}$  is much less soluble than  $\text{Fe}^{2+}$  at pH 5, Reactions 7 and 8 occur:



Reaction 7 produces the typically acidic reaction at an anodic location. In this localized anodic area, the pH of the solution is much lower than that of the bulk solution.

The chloride requirements for stress corrosion cracking of stainless steel in the reactor system can be expressed quantitatively. A crack can be initiated in sensitized, stressed, stainless steel at an anodic location by  $5 \times 10^{-9}$  g  $\text{Cl}^-$ , as shown by Overman.<sup>13</sup> Localized dissolution of 10  $\mu\text{g}$  of alumina containing 500 ppm  $\text{Cl}^-$  could provide enough chloride at an anodic site to cause cracking, if all the chloride were adsorbed. Although it is unlikely that all the chloride would be adsorbed, the negative charge on the chloride would cause the chloride to be attracted to the positively charged anodic area. If only 50% of the chloride released from dissolving alumina were absorbed at an anodic site, a crack would require dissolution of either 10  $\mu\text{g}$  of alumina containing 1000 ppm  $\text{Cl}^-$  or 20  $\mu\text{g}$  of the originally assumed concentration. These assumed chloride concentrations have often been found in Savannah River reactor systems.

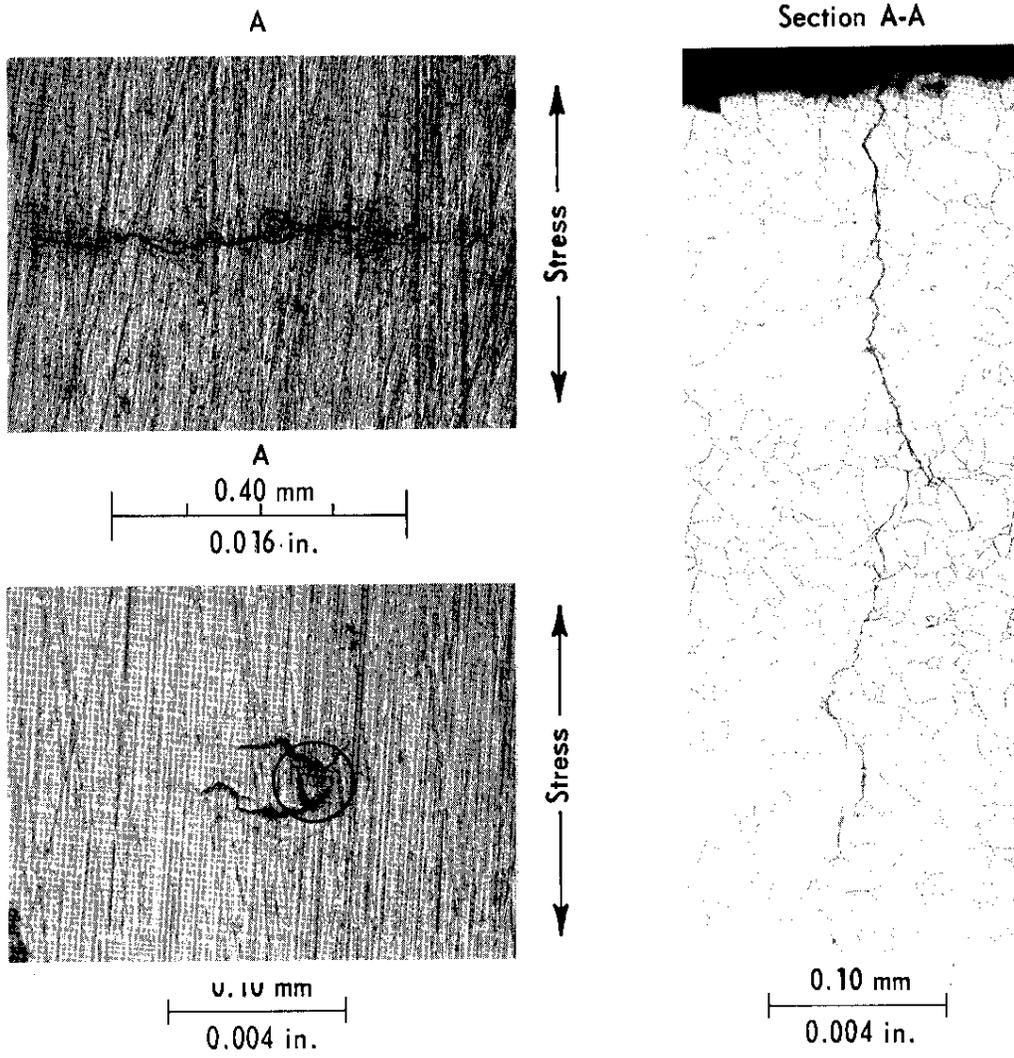


FIG. 8 CRACKING OF 304 STAINLESS STEEL BY CHLORIDE-BEARING ALUMINA

Initial anodic areas circled where chloride was released due to acid formation in pH 7 solution. Note initial intergranular and final transgranular failure, similar to cracks observed in reactor system.

## MINIMIZING STAINLESS STEEL CRACKING

Deposition of chloride-bearing alumina is indicated to be a probable factor in causing occasional stress corrosion cracking of stainless steel that has been sensitized, pickled, and ground.

Based on this mechanism, sensitized metal will continue to crack at anodic sites as long as chloride-bearing alumina remains in the coolant system.

This type of cracking can be minimized by

- o Maintaining the pH (pD) of the moderator-coolant between 5.0 and 5.2.
- o Eliminating the use of chlorinated hydrocarbons as degreasing agents for aluminum to avoid addition of chloride to the reactor.
- o Deactivating the existing anodic sites by changing the stainless steel potential, such as by installing sacrificial anodes or by adding inhibitors to the moderator.
- o Removal of chloride-bearing alumina from the system.

Control of pD is already a standard procedure in normal operation of SRP reactors; practical and feasible methods for accomplishing the remaining items have not yet been established.

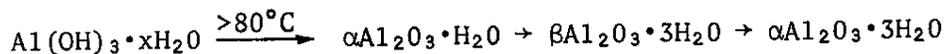
## APPENDIX

### EXPERIMENTAL PROCEDURES

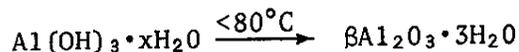
#### Preparation of Amorphous and Crystalline Aluminas

Amorphous alumina,  $\text{Al}(\text{OH})_3 \cdot x\text{H}_2\text{O}$ , was prepared by corrosion of amalgamated aluminum in water at  $25^\circ\text{C}$ . The specimens of aluminum were first cleaned in 1M NaOH to remove oxide, rinsed, and dipped in a 0.1M mercuric nitrate solution until the metal was coated with mercury. The specimen was then rinsed for several minutes and dipped in deionized water to corrode.

Amorphous alumina gradually transforms to crystalline alumina; the crystal form finally obtained is governed by the temperature maintained during the transformation. The  $\alpha$  trihydrate forms by the following reactions:



The  $\beta$  trihydrate forms as follows:



X-ray diffraction was used to confirm transformation to the desired form.

Radioactive  $\alpha$  and  $\beta\text{Al}_2\text{O}_3 \cdot 3\text{H}_2\text{O}$  were prepared by corroding aluminum in water containing 5000 ppm  $\text{Cl}^-$  with  $^{36}\text{Cl}$  tracer.

Solids containing  $^{36}\text{Cl}$  were counted for beta activity by a GM counter. Solutions containing  $^{36}\text{Cl}$  were counted by liquid scintillation techniques.

#### Preparation of Stainless Steel Specimens

Specimens of 304 stainless steel  $3/4 \times 3 \times 0.060$  in. thick were sensitized by heating to  $625^\circ\text{C}$  for 1 hour. The specimens were then pickled by swabbing with 1M HF-4M  $\text{HNO}_3$  to remove the oxide film, rinsed, and dry ground with a 60-grit wheel. Four-point loading jigs were used to stress the specimens to a calculated 75% of yield, outer fiber stress, at room temperature.<sup>14</sup>

## REFERENCES

1. S. P. Rideout. "Stress-Corrosion Cracking of Type 304 Stainless Steel in High-Purity Heavy Water." *Proceedings of 2nd International Congress on Metallic Corrosion in New York City, March 11-15, 1963.* pp. 159-71, National Association of Corrosion Engineers, Houston (1966).
2. R. H. McKane, H. C. Minton, Jr., and J. W. Wade. *Stainless Steel Failures in Savannah River Plant Reactor Areas*, USAEC Report DP-539, E. I. du Pont de Nemours & Co., Savannah River Laboratory, Aiken, S. C. (1960).
3. J. P. Maloney et.al., *Repair of a Nuclear Reactor Vessel*, USAEC Report DP-1199, E. I. du Pont de Nemours & Co., Savannah River Laboratory, Aiken, S. C. (1969).
4. H. H. Uhlig. *Corrosion and Corrosion Control.* p 276, John Wiley and Sons, New York (1963).
5. R. M. Latanision and R. W. Staehle. "Stress Corrosion Cracking of Iron-Nickel-Chromium Alloys." *Fundamental Aspects of Stress Corrosion Cracking.* pp. 214-307, National Association of Corrosion Engineers, Houston (1969).
6. B. E. Wilde. "Technique for Studying the Kinetics of Intergranular Crack Nucleation on AISI Type 304 Stainless Steel in Oxygenated Water at 289 C." *Corrosion* 25, 359 (1969).
7. M. Pourbaix. *Atlas of Electrochemical Equilibria in Aqueous Solution.* p. 174, Pergamon Press, Ltd., New York (1966).
8. K. Funaki and Y. Shimizu. "The Properties of Hydrated Alumina and Alumina." *J. Chem. Soc. Japan* 55, 194 (1962).
9. B. Verkerk. "Autoradiographic Study of the Influence of Chlorine on Semi-Conducting Alumina Layers." *Radioisotopes in Scientific Research, Proceedings of the First (UNESCO) International Conference 1*, 503-515 (1957).
10. J. E. Draley and W. E. Ruther. "The Corrosion of Aluminum Alloys in High Temperature Water." *Corrosion of Reactor Materials 1*, p. 477, International Atomic Energy Agency, Vienna (1962).
11. R. L. Dillon and H. C. Bowen. "A Basis for Design of Aluminum Alloys for High Temperature Water Service." *Corrosion* 18, 406t-16t (1962).

12. Ketil Videm. *Aluminum Alloys with Improved Corrosion Resistance in High Temperature Water*. Norwegian Report KR-19, Institute for Atomenergi, Kjeller Research Establishment, Lillestrom, Norway (1962).
13. R. F. Overman. "Using Radioactive Tracers to Study Chloride Stress Corrosion Cracking of Stainless Steels." *Corrosion* 22, 48 (1966).
14. L. L. Shreier. *Corrosion*. Vol. 2, pp. 20-59, John Wiley and Sons, New York (1963).

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Author: R. S. Ondrejcin

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January 28, 1970

Mr. A. F. Westerdahl, Chief  
Patent Branch  
Savannah River Operations Office  
U. S. Atomic Energy Commission  
Post Office Box A  
Aiken, South Carolina 29801

Dear Mr. Westerdahl:

REQUEST FOR PATENT REVIEW

Please review for patent matter:

DP-1089 "A Mechanism for Stress Corrosion Cracking  
of Stainless Steel in Reactor Systems" by  
R. S. Ondrejcin

If any technical clarification is needed please call J. E. Beach whose  
document review is attached.

Please telephone your comments to the TIS Office (Ext. 3598) and notify me  
by signing and returning to TIS the original of this letter. A copy is  
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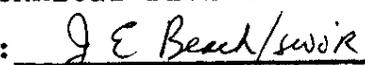
If you decide to pursue a patent on any development covered, I shall be happy  
to supply additional information required such as appropriate references and  
the names of persons responsible for the development.

The above item is approved  
for release.

Very truly yours,

  
A. F. Westerdahl Date 3/2/70  
Chief, Patent Branch  
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J. W. Croach, Director  
Technical Division

By:   
J. E. Beach



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*ETH Comments?*  
*SM*

February 16, 1970

R. S. Ondrejcin  
Savannah River Laboratory  
Building 773-A

DP-1089  
A MECHANISM FOR STRESS CORROSION CRACKING  
OF STAINLESS STEEL IN REACTOR SYSTEMS

The preliminary copy of this report has recently been released. It concerns an investigation of the role of chloride ion in the stress-corrosion of stainless steel which is covered with deposits of alumina. The work was conducted several years ago. Stress corrosion of stainless steel was demonstrated in the laboratory under conditions in which the sole source of chloride ion was that absorbed in alumina which was applied to the surface.

In the past several years it has become evident from nuclear power plant experience that oxygen in coolant streams is uniquely responsible for the corrosion, by intergranular attack, of susceptible stainless steels. The descriptions of the plant conditions under which such corrosion was observed, supplemented by the published details of a number of laboratory investigations of these problems strongly suggest that our corrosion may also be due to the presence of oxygen (and peroxides) in our systems. Corrosion occurs in the power reactor systems at about 300°C (and in relatively shorter time periods). With this exception, the characteristics of the power reactor corrosion appear to be identical to ours:

1. Corrosion originates on the process (moderator) side.
2. The attack is exclusively intergranular (IGA). *Not true!*

*Also, do have rust spots that are normally not observed in BWR's.*

February 16, 1970

3. Only severely sensitized wrought stainless steel is involved. Neither mildly sensitized wrought material nor weld metal is attacked.
4. Intergranular attack of susceptible material occurs almost exclusively in BWR's (in which oxygen occurs at approximately 0.1 ppm). It has occurred in PWR's (which have hydrogen blankets) only in blind spots in which the atmosphere was inadvertently out of control.
5. Both BWR & PWR's have  $\text{Cl}^-$  controlled to less than 0.1 ppm.

As agreed in our telephone conversation I am enclosing photocopies of relevant literature on the role of oxygen. With this material in hand it was agreed that you would expand your report to the point that it represents current evidence in this field.

ATOMIC ENERGY DIVISION

*W. B. DeLong*William B. DeLong, Director  
Nuclear Materials Section

WBD/ekh

Attachment

BIBLIOGRAPHY

1. B. E. Wilde & J. E. Weber, Intergranular Stress-Corrosion Resistance of Austenitic Stainless Steels in Water/Oxygen Environments: Accelerated Test Procedure  
British Corrosion Journal, Vol. 4, p 42, 1969
2. B. E. Wilde, Technique for Studying the Kinetics of Intergranular Crack Nucleation on AISI Type 304 Stainless Steel in Oxygenated Water at 289°C  
Corrosion, Vol. 25, p 359, 1969
3. J. S. Armijo, Intergranular Stress Corrosion Cracking of Austenitic Stainless Steels in Oxygenated High Temperature Water  
Corrosion, Vol. 24, p 319, 1968
4. L. R. Scharfstein & W. F. Brindley, Chloride Stress Corrosion Cracking of Austenitic Stainless Steel-Effect of Temperature and pH  
Corrosion, Vol. 14, p 588t, 1958
5. T. J. Pashos, W. H. Arlt, C. J. Baroch & H. E. Williamson, The Intergranular Corrosion of Stainless Steel Cladding in Water Reactor Environments  
Trans. Am. Nucl. Soc., Vol. 7, p 119, 1964
6. D. Birchon & G. C. Booth, Stress Corrosion Cracking of Some Austenitic Steels as Affected by Surface Treatment and Water Composition  
2nd Int. Congress on Metallic Corrosion
7. W. E. Berry, Some Facts About Stress Corrosion of Austenitic Stainless Steels in Reactor Systems  
Reactor Mat'ls., Vol. 7, p 1-13, 1964
8. D. A. Vaughan, D. I. Phalen, C. L. Peterson & W. K. Boyd, Relationship Between Hydrogen Pickup and Susceptible Paths in Stress Corrosion Cracking of Type 304 Stainless Steel  
Corrosion 19, p 315t, 1963
9. P. D. Neumann & J. C. Griess, Stress-Corrosion Cracking of Type 347 Stainless Steel and Other Alloys in High Temperature Water  
Corrosion, Vol. 19, p 345t, 1963

10. H. R. Copson & G. Economy, Effect of Some Environmental Conditions on Stress Corrosion Behavior of Ni-Cr-Fe Alloys in Pressurized Water  
Corrosion, Vol. 24, p 55, 1968
11. R. H. Holyoak, G. R. Forrer, J. P. Higgins, In-Service Inspection of Dresden-1  
Nuclear News, p 32, Nov. 1968
12. Oyster Creek Nuclear Power Plant, Unit No. 1, Appendix A, Amendment 37, Response to AEC Letter of April 15, 1968
13. L. H. Roddis, Jr., Nuclear Materials, Paper presented at Brooklyn Conference on Material Considerations in Nuclear Power Plant Applications, N.Y.C. - 10/30/68
14. L. E. Lesurf, P. E. C. Bryant & M. G. Tanner, Use of Ammonia to Suppress Oxygen Production and Corrosion in Boiling-Water Reactors  
Corrosion, Vol. 23, p 57, 1967
15. C. F. Cheng, Corrosion Aspects of Iron, Austenitic Stainless Steel and Inconel in High Temperature Water  
Corrosion, Vol. 20, p 341t, 1964
16. D. C. Vreeland, G. G. Gaul & W. L. Pearl, Corrosion of Carbon Steel and Other Steels in Simulated Boiling-Water Reactor Environment: Phase II  
Corrosion, Vol. 18, p 368t, 1962
17. R. N. Stanutz, D. R. McClintock, W. S. Hazelton, The Behavior of Sensitized Stainless Steel in a Pressurized Water Reactor  
Am. Nucl. Soc. Transactions, Vol. 9, p 33, June 1966
18. Pertinent SRP, SRL Reports  
DP 539  
DPSPU 61-11-4  
DPSPU 62-11-17  
DPSPU 62-11-26  
DPSPU 64-11-4  
DPSPU 67-11-4